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# **Project SAFE**

# Radionuclide release and dose from the SFR repository

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October 2001

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This report concerns a study which was conducted for SKB. The conclusions and viewpoints presented in the report are those of the author(s) and do not necessarily coincide with those of the client.

## Summary

The objective of this report is to describe the radionuclide release and dose calculations for the SFR 1 repository within the SAFE study.

A number of calculation cases for quantitative analyses have been defined based on the expected development of the conditions in the repository, rock and biosphere for a base scenario and for other scenarios.

In this study three cases are analysed within the base scenario to illustrate the expected development in the near field, i.e. the vaults and Silo including the waste and technical barriers. The main case with intact barriers, a variation case with degraded technical barriers after 1000 years and a variation case with influence of gas. The variation case with influence of gas is presented in Moreno et al. (2001). The influence of the surrounding rock on the release of radionuclides is illustrated with two cases; included or neglected geosphere. Changes in discharge points and ecosystems in the biosphere are studied by calculation of human dose for a reasonable biosphere development, two variations of the development of the ecosystem (mire and well) as well as today's biosphere during the whole time period. The reasonable case assumes that the release of radionuclides from the time for repository closure until 5000 AD is to a coast recipient that corresponds to a part of the present Öregrundsgrepen. The ongoing land rise will decrease the size of Öregrundsgrepen with time. For the time period 4000 AD to 5000 AD the water volume and water turn over is thus assumed to be less than today. At 5000 AD the radionuclides are released to a lake that is developed in the area. This lake will remain as recipient until 8000 AD where after the radionuclides are assumed to be released to agricultural land until 12 000 AD.

In addition to the base scenario, calculations are performed for some other scenarios. The other scenarios include; initially degraded barriers, influence of chemicals, a combined effect of degraded barriers and influence of chemicals, permafrost and human intrusion.

The result for the base scenario with intact barriers and a reasonable biosphere development is an initial dose of about  $10^{-8}$  Sv/yr determined by  $^{137}$ Cs released from BLA. The maximum total dose for the whole SFR 1 repository is  $4 \cdot 10^{-6}$  Sv/yr at 5000 AD totally dominated by organic  $^{14}$ C from the Silo. The maximum dose if the release occurs to a mire area is  $6 \cdot 10^{-6}$  Sv/yr. The dose is initially dominated by  $^{239}$ Pu and  $^{240}$ Pu from BLA and later by  $^{79}$ Se. The maximum dose if the release occurs to a well with a recharge area from all vaults is  $4 \cdot 10^{-4}$  Sv/yr. The dose is totally dominated by  $^{239}$ Pu and  $^{240}$ Pu from BLA. The alternative well with a recharge area from the Silo results in a maximum dose of  $5 \cdot 10^{-5}$  Sv/yr, dominated by organic  $^{14}$ C. In the case with release to today's biosphere the maximum dose is obtained directly after repository closure from release of  $^{137}$ Cs from BLA at a level of  $10^{-8}$  Sv/yr.

The case with degraded technical barriers results in a maximum dose for SFR 1 for all studied biospheres that is up to a factor three higher than that for intact barriers. The largest increase in dose is obtained for release from the Silo to mire.

## Sammanfattning

Syftet med denna rapport är att beskriva radionuklidtransport- och dosberäkningarna för SFR 1 inom SAFE studien.

För den kvantitativa analysen har ett antal beräkningsfall definierats baserat på den förväntade utvecklingen av förhållandena i förvaret, omgivande berg och biosfären för ett basscenario och för övriga scenarier.

I denna studie analyseras tre fall inom basscenariot för att illustrera den förväntade utvecklingen i närzonen, bergsalarna och Silon inklusive avfall och tekniska barriärer. Huvudfallet med intakta barriärer, ett variationsfall med degraderade tekniska barriärer efter 1000 år och ett variationsfall med inverkan av gas. Variationsfallet med inverkan av gas presenteras i Moreno et al. (2001). Inverkan av det omgivande berget på radionuklidutsläppet illustreras i två beräkningsfall, inkluderad eller försummad geosfär. Förändringar i utsläppspunkt och ekosystem i biosfären studeras i beräkningarna av dos till människa för en rimlig biosfärsutveckling, två alternativa utvecklingar av ekosystemet (myr och brunn) liksom dagens biosfär för hela tidsperioden. I det rimliga fallet antas att utsläppet av radionuklider från förvarets förslutning till 5000 AD sker till en kustrecipient som motsvarar en del av den nuvarande Oregrunsgrepen. Den pågående landhöjningen medför att storleken på Öregrundsgrepen minskar med tiden. För tidsperioden 4000 AD till 5000 AD antas därför att vattenvolymen och vattenomsättningen är mindre än idag. Det antas vidare att en sjö har bildats i området och att denna sjö utgör recipient för utsläpp av radionuklider mellan 5000 ADoch 8000 AD. Från 8000 AD till och med 12 000 AD släpps nukliderna ut till ett jordbruksområde.

Beräkningar har förutom för basscenariot genomförts för ett antal övriga scenarier. Dessa scenarier omfattar initialt degraderade barriärer, inverkan av kemikalier, en kombinerad effekt av degraderade barriärer efter 1000 år och kemikalier, permafrost och slutligen ett intrångsscenario.

Resultatet för basscenariot med intakta barriärer och en rimlig biosfärsutveckling är en initial dos på ungefär 10<sup>-8</sup> Sv/år som domineras av utsläpp av <sup>137</sup>Cs från BLA. Den totala maximaldosen för utsläpp från hela SFR 1 är 4·10<sup>-6</sup> Sv/år vilket erhålls år 5000 AD. Dosen domineras totalt av utsläppet av organiskt <sup>14</sup>C från Silon. Om utsläppet istället sker till ett myrområde erhålls en maximal dos på 6·10<sup>-6</sup> Sv/år. Dosen domineras till en början av <sup>239</sup>Pu och <sup>240</sup>Pu från BLA och senare av <sup>79</sup>Se. Utsläpp av radionuklider från de fyra bergsalarna till en brunn ger som mest en totaldos på 4·10<sup>-4</sup> Sv/år, vilken domineras helt och hållet av utsläpp av <sup>239</sup>Pu och <sup>240</sup>Pu från BLA. Motsvarande utsläpp från Silon ger en maximal totaldos på 5·10<sup>-5</sup> Sv/år dominerad av organiskt <sup>14</sup>C. Vid utsläpp till dagens biosfär erhålls den maximala dosen omedelbart efter förvarets förslutning. Den maximala dosen är ungefär 10<sup>-8</sup> Sv/år och domineras av <sup>137</sup>Cs från BLA.

Beräkningsfallet med degraderade tekniska barriärer ger en maximal dos för hela SFR 1 för alla studerade biosfärer som är upp till tre gånger högre än för fallet med intakta barriärer. Den största ökningen i dos erhålls för utsläpp från Silon till en myr.

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# 1 Introduction

### 1.1 Background

Differences between this study and older studies of SFR 1 that influences the calculations and results of radionuclide release are:

The waste inventory has been updated, based on new detailed information of produced and planned waste (Riggare and Johansson, 2001). The total activity content is unchanged,  $10^{16}$  Bq, but the distribution between different nuclides is different. The radionuclide inventory is given in detail, i.e. in each waste package instead of in each repository part only. This implies a possibility to describe the radionuclide release better.

New hydrogeological calculations (Holmén and Stigsson, 2001a) result in a more detailed description of the water flow inside the repository parts. The water flow rates inside the repository are lower in this study, since they were chosen very pessimistic in the older studies due to the lack of detailed information. Although the new near-field calculations are complex and many input parameters are changed the decrease in water flow is one of the main reasons to the lower near-field release rates for most calculation cases compared to the previous studies.

All input data (SKB, 2001a) have been thoroughly reviewed and many small changes were performed compared to older studies. Of major importance in the near field is decreased sorption capacity for nickel.

The near-field model set up to describe the release is made more complex, to be able to include the more detailed information on the inventory and water flow. This model may be seen as a somewhat simplified three-dimensional model. Older studies (Wiborgh *et al.*, 1987; Lindgren and Pers, 1991) were performed with a model describing different release path separately. The discretizations were finer in the transport direction in the older models. The performed changes imply a better description of influence between different release paths but a poorer description of the instationary diffusion.

In this study far-field calculations were performed for all cases. In older studies simplified calculations for selected nuclides have been performed (Neretnieks *et al.*, 1987).

For the biosphere calculations the most evident difference from earlier studies is the construction of a case where the ecosystems in the area changes with time as a consequence of the land rise (here called reasonable biosphere development). This approach also includes later releases of radionuclides accumulated in sediments. In the earlier study two separate cases – coast and inland – were modelled, without interaction

## 1.2 Objective

The objective of this report is to describe the radionuclide release and dose calculations for the SFR 1 repository within the SAFE study. The information used in the calculations is documented and the results are presented.

The aims of the radionuclide release calculations are to quantitatively describe the radionuclide transport and to analyse the effect of different scenarios on the environment during 10 000 years after repository closure, both in the near field and biosphere.

By performing several calculation cases for each repository part, enough knowledge and understanding are achieved to fulfil these aims.

#### 1.3 Structure of report

The different calculation cases are described in Chapter 2 and the computer codes and the compartment models in Chapter 3. The input data used are summarised in Chapter 4. The results for the near field, far field and biosphere are presented in Chapter 5 for the base scenario and in Chapter 6 for the other scenarios. The base scenario comprises a main case and a case with degraded barriers. Other scenarios treat influence of chemicals, permafrost and human intrusion. A summary of the results is presented in Chapter 7.

## 2 Description of calculation cases

#### 2.1 Choice of calculation case

A number of calculation cases for quantitative analyses have been defined based on the expected development of the conditions in the repository, rock and biosphere for the base scenario and for the other scenarios. The expected evolution of the repository system, the scenario analysis and the selection of calculation cases are given in the scenario report (SKB, 2001b). The aim is that the calculation cases will consider the expected development and the uncertainties connected to it, both concerning processes and events that influences the state of the system and quantitative data that describe these states. The uncertainties are illustrated by analysing different calculation cases for the base scenario in combination with pessimistic choice of the parameter values used in the calculations.

A schematic description of the calculation cases is shown in *Figure 2-1* and a short summary is given in Table 2-1. Three cases are analysed within the base scenario to illustrate the expected development in the near field, i.e. the vaults and Silo including the waste and technical barriers. The influence of the surrounding rock on the release of radionuclides is illustrated with two cases; included or neglected geosphere. Changes in discharge points and ecosystems in the biosphere are studied by calculation of human dose for a reasonable biosphere development, two variations of the development of the ecosystem as well as today's biosphere during the whole time period.



Figure 2-1. Schematic description of the calculation cases.

Scenario/Calculation	Description					
case						
Base scenario						
Main case (intact barriers)	All technical barriers fulfil the specifications of the construction at closure. Constant properties during the whole time period. Water flow in the near field influenced by the ongoing land rise. Dose to human for a reasonable biosphere development that imply release of radionuclides to Öregrundsgrepen during the time period 2000 AD to 5000 AD, release to a lake during the period 5000 AD to 8000 AD and release to agricultural land during the period 8000 AD to 12 000 AD. Dose consequence also for the alternatives with release of radionuclides to a well downstream the repository from 4000 AD to 12 000 AD, release of radionuclides to a mire area from 4000 AD to 12 000 AD and for release to Öregrundsgrepen with today's conditions during the whole time period.					
Influence of gas	Expelled water containing radionuclides from the near field barriers due to gas produced in the near field. Other conditions as in the case "intact barriers". (not included in this report, see Moreno <i>et al.</i> , 2001)					
Degraded technical barriers	Conditions as in the case "intact barriers" during the time period 2000 AD to 3000 AD. Fractured/degraded barriers in the near field implies changed flow conditions in the near field from 3000 AD until 12 000 AD. Other conditions in the near field, geosphere and biosphere as in the case "intact barriers".					
Other scenario: Initial defects						
structure in the concrete	increase the water flow through the technical barriers in comparison to the case "intact barriers" in the Base scenario. Other conditions as in the case "intact barriers".					
Fracture in the concrete structure and gas	Water containing radionuclides expelled through fracture(s) in the concrete barriers due to gas produced in the near field. Other conditions as in the case " Fracture in the concrete structure". (not included in this report, see Moreno <i>et al.</i> , 2001)					
Chemicals/complexing agents	Considerably poorer sorption in the technical barriers in the near field. Other conditions as in the case "intact barriers" in the Base scenario.					
Chemicals/complexing agents and degraded barriers	Considerably poorer sorption in the technical barriers in the near field. Other conditions as in the case "degraded technical barriers" in the Base scenario.					
Other scenario: Climate						
Permafrost	Frozen repository until 12 000 AD when radionuclide release occurs with the total flow through the different repository parts. Dose consequence with today's biosphere, Öregrundsgrepen.					
Other scenario: Human activi	íty					
Well in the repository	A well is sunk in the repository earliest when the shoreline passes by the repository area, i.e. earliest at 3000 AD. Immediate dose consequence from ingestion. The long term dose consequence caused by the well are changed flow conditions, but otherwise same conditions as in the case "intact barriers" in the Base scenario.					

 Table 2-1.
 Summary of the calculation cases.

#### 2.2 Base scenario - Near field

The aim of the calculations for the base scenario is to show the ability of the repository to prevent radionuclide release from the waste, taking the future changes in barrier properties and other conditions into consideration. To fulfil this purpose a main case and

two variation cases have been defined. The purpose of the variation cases is to illustrate the consequence of possible but more uncertain and not in detailed studied processes. One variation case show the consequence of water containing radionuclides expelled by gas formed in the repository. The other case shows the influence of an extensive degradation of the technical barriers with time. A general description of transport processes, barrier properties and other conditions taken into consideration in the different cases is given below. A more detailed description is given in the report describing the scenario and system analysis (SKB, 2001b).

#### 2.2.1 Main case - intact barriers

One of the prerequisites of the base scenario is that the properties of all technical barriers fulfil the requirements at repository closure. This implies that concrete floor, walls and lid in the different repository parts do not contain large intersecting fractures. The same concerns the porous concrete or concrete grout surrounding the waste packages. Small fractures in the concrete may be formed due to stress in the material. Bentonite and sand/bentonite barriers in the Silo are assumed to be homogeneous.

The concrete moulds and concrete tanks are supposed to be almost intact at closure, i.e. they do not contain large intersecting fractures. Steel packagings may very well be tight at closure, but there might also be damages caused during the operational period, for example due to corrosion. Steel packagings are therefore not considered as barriers for water and dissolved radionuclides.

After closure groundwater will flow into the different repository parts from the surrounding rock and fill up pore volumes and empty space in the repository. The time from repository closure to a water filled repository is rather short, some few years for the vaults and some ten years for the Silo (Holmén and Stigsson, 2001a). In the calculations the time to fill the repository is neglected and a water filled repository is assumed to prevail immediately after repository closure.

When water flows into the different repository parts and come into contact with the waste the radionuclides in the waste dissolves in the water. Limitations in the solubility and availability in the waste will delay the dissolution and thereby influence the concentration of radionuclides in the water in contact with the waste. Dissolved radionuclides are thereafter transported through the waste matrix, walls of waste packages and surrounding concrete, bentonite and gravel barriers out from the different repository parts. The transport occurs by diffusion and with the water that flows in the different barriers but the release is delayed by sorption in the barrier materials. The quantity of the radionuclides that are released from the different repository parts is thus dependent on the concentration of radionuclides in the water in contact with the waste, on the diffusion and sorption properties of the barrier materials and on the size and distribution of the water flow in the barriers and how these change with time.

No limits in solubility or availability to dissolve in the water in contact with the waste are taken into account for the nuclides. However, the calculations take into account a certain time for the release of radionuclides in waste stabilised in bitumen. The release of radionuclides from a bitumen matrix is insignificant unless a network of pores or fractures is formed in the matrix. Such a network can be formed by several mechanisms (Pettersson and Elert, 2001). Once an opening to the surface of the matrix is established the radionuclides dissolve in the water filled pores or fractures and are released from the matrix by diffusion. This is a complicated course of events that is difficult to quantify, but a conservative estimate is that it will take at least 100 years before all nuclides in a bitumen matrix are dissolved. In the calculations it is therefore assumed that one percent of the initial radionuclide content in the waste is released from the bitumen matrix every year.

The resistance to diffusion and flow in the barriers depends on their porosity as well as the presence of fractures in the materials. Due to chemical reactions between the barrier material and components in the water these properties will change with time. Analysis performed indicate only small changes in the concrete barriers, even after long time (Höglund, 2001). In the calculations it is assumed that there are no changes of the fractures or porosity with time. Instead, values chosen on porosity, diffusion coefficient and hydraulic conductivity represent materials with the expected increase in porosity with time and small fractures in the concrete barriers already from repository closure.

The properties of the bentonite barriers will also change with time due to chemical reactions and the possibility exists that the concrete barriers may crack due to mechanical impact, from for example increased pressure from gas production. These mechanisms have not been studied in detailed within the SAFE project. Possible consequences of this type of degradation are difficult to define but are illustrated separately as a variation case, fractured/degraded barriers, to the main case.

Size and direction of the water that flows through the different repository parts and their barriers will change with time even if the hydrological properties are constant. This is due to the land rise and its influence on the groundwater flow in the surrounding rock. In the calculations this is simulated by stepwise changes of size and direction of the flow in accordance with the results from the hydrogeological calculations in the detailed model (Holmén and Stigsson, 2001a). Size and direction of the flow calculated for the time 2000 AD is assumed to be valid from repository closure to 3000 AD. At this time the size and direction of the flow are changed according to the hydrogeological calculations at 3000 AD. Stepwise changes are performed in the same way at 4000 and 5000 AD but since only small changes are expected for longer times stationary flow conditions is assumed to prevail from 5000 AD and ahead.

The transport of radionuclides in the near field will be delayed by sorption in cement and concrete as well as in bentonite barriers and gravel backfill. In cement and concrete barriers the penetrating water will quickly obtain a high pH, and the ion strength will be high since the penetrating water is salt and concrete contribute with dissolved salts. With time the leakage of cement components may result in some decrease in pH and ion strength in cement and concrete barriers, but alkaline conditions will be maintained during long time. The expected decrease in pH is not judged to influence the sorption in the cement and concrete barriers negatively but decreased ion strength will increase the sorption of elements like caesium and strontium that sorb by ion exchange. Therefore the sorption data is chosen to be representative for fresh cement and concrete with a high pH and high ion strength for the whole time period.

The water in the bentonite barriers and gravel backfill will at repository closure have pH and ion strength more in accordance with the penetrating groundwater. Leaching of concrete components will give rise to an increase in pH with time and also change the chemical composition of the solid materials. These processes have not been analysed, but they are not expected to decrease the sorption in comparison with the sorption in these materials at repository closure. The reason for that is that an increase in pH favours the sorption and that the secondary minerals formed in contact with alkaline water are mostly good sorbents. Therefore sorption data for gravel backfill and bentonite barriers representative for a saline groundwater are chosen and these are assumed to be valid for the whole studied time period. The future change from saline to

non-saline groundwater is neglected since this only implies sorption of elements that sorb via ion exchange and thereby increase the sorption capacity for them.

Products from corrosion of iron and steel are not taken into account as sorption barrier despite that there is strong evidence that iron oxides and iron hydroxides bind many elements. Reported sorption data in the literature given as Kd-values are for many nuclides as high and in some cases higher than for sorption on cement and concrete (Savage *et al.*, 2000). Sorption on corrosion products can thus be of importance for the release of radionuclides from the near field, especially from BLA that does not have concrete barriers.

The waste contains chemicals that may form complex with the radionuclides and thereby influence the sorption of the radionuclides. Cellulose in the waste and as additive in cement and concrete may through alkaline degradation form isosaccarinic acid (ISA) that is a strong complexing agent. Based on estimated amounts of chemicals and cellulose materials in different waste types in SFR 1 the concentrations of complexing agents inside the waste packages have been calculated (Fanger *et al.*, 2001). This shows that the concentration of complexing agents inside the waste packages for a few waste types can be so high that it can not be disregarded that sorption inside the packages with bitumenised waste in steel packaging, but these are, however, not treated as a sorption barrier anyway.

Fanger and co-workers also show that there are a few waste types with cement stabilised waste where the concentration of ISA is of the same level as the threshold concentration  $(1 \cdot 10^{-4} \text{ M})$  where ISA may lead to a small reduction in sorption of fourvalent and pentavalent elements in the cement matrix. Any influence on the sorption of these elements in mould walls or other concrete barriers outside the cement matrix at this low concentration of ISA seems not especially likely. The reason for this is that the ISA formed will to a large extent be kept inside the cement matrix since ISA itself sorb on cement. In addition there are experiments that indicate that lower concentrations of ISA do not influence sorption of elements in complexes with ISA.

There are also chemicals within the waste that may act as complexing agents. Two of these (EDTA and sodium capryliminodipropionat, NKP) give concentrations inside the waste packages of a few waste types that may imply a reduction of the sorption of Ni and chemically analogue divalent elements in the waste matrix with cement.

Regarding the expected small effects from complexing agents on the sorption and that the data representing the sorption in the system without complexing agents are already conservatively chosen, the possible effects from complexing agents are neglected in the base scenario. The presence of high concentrations of complexing agents may however be of importance for the safety of the repository. Thus this is illustrated as an initial defect scenario where the presence of complexing agents result in a considerable reduction of the sorption in the near field barriers. This case is further described in Section 2.5.1.

Despite complexing agents, colloids in water may reduce the delay of radionuclides in the near field barriers. Radionuclide transport with colloids in the near field is neglected due to the expected low concentrations of colloids in near-field water with such high concentrations of salt.

#### 2.2.2 Fractured/degraded barriers

The performed analysis on the future chemical degradation of the concrete barriers in SFR 1 show only small changes even after long times (Höglund, 2001). There are, however, other processes that have not been studied in detail that may influence the barrier properties in the long term. Concerning cement and concrete barriers it is mainly mechanisms that can change the transport properties by causing fractures that need to be considered. Examples of such mechanisms are pressure build-up due to gas production, pressure from expanding corrosion products and stress in the materials caused by settlements and movements in the system. These types of changes can increase the hydraulic conductivity in cement and concrete barriers and also form preferential transport paths for the radionuclides.

The properties of bentonite barriers can also change, especially through chemical reactions with cement components. The performed analysis that has been made shows that rather fast ion exchange may occur in sand/bentonite. Chemical transformations of bentonite caused by high pH are also possible. This type of chemical transformations can imply reduced hydraulic properties in the bentonite barriers with time. The sorption properties are probably not negatively influenced since the minerals that can be formed, for example CSH phases, are as good sorbents as the original minerals.

The effect of deteriorating hydraulic properties with time in cement and concrete barriers as well as in bentonite barriers is illustrated in a variation case within the base scenario. Since the lapse of time for the degradation of the barriers is unknown it is assumed that the degeneration occur momentarily 1000 years after repository closure, i.e. at 3000 AD.

For the Silo the same conditions are assumed as in the base case despite other water flows through the Silo barriers from 3000 AD and further on. The size and direction of this water flow from 3000 AD and further on are given by the hydrogeological modelling in the detail scale with increased hydraulic conductivity in the barriers compared to the main case (Holmén and Stigsson, 2001a).

Also for BMA all conditions are assumed to be the same as in the main case except changed water flows from 3000 AD and further on. At this point the concrete walls in one room, room 12, closest to the fracture zone 6, are degraded. The size and direction of the water flow in different parts of BMA with degraded walls was calculated with the hydrogeological model in the detail scale and the results for year 3000 AD and further on are used in the calculations of the radionuclide release from the near field.

No calculations are performed for the radionuclide release from the BTF vaults in the case with degraded barriers after 1000 years. However an estimate is performed on possible changes based on calculated changes of the water flow.

#### 2.3 Base scenario - Geosphere

Radionuclides transported out from the technical barriers in the different repository parts are transported further in the rock by the flowing water. The transport occurs mainly by advection in the open fractures in the rock. The ability of the rock to delay and thereby through decay reduce the release of radionuclides to the recipients in the biosphere compared to the release from the near field depend upon the so called transport resistance (see for example Andersson *et al.*, 1998). The transport resistance is determined by the size of the groundwater flow, the travel distance and the exchange of radionuclides between the flowing water and the rock matrix.

The ongoing land rise displaces the shoreline and thus both the travel distance and the groundwater flow change with time. The effect of that on the transport and delay of radionuclides are, however, relatively limited. Two simplified cases are illustrated.

In one case, it is assumed that the release occur directly from the near field to the recipient in the biosphere. This pessimistic case is based on the expected low transport resistance during the first 1000 to 2000 years after repository closure as well as uncertainties in parameters that quantifies transport and processes that may reduce travel times, for example transport with colloids and microbes.

In the other case the time dependence of the transport resistance is neglected and the influence of the rock on the release of radionuclides is illustrated with constant values on the migration parameters included in the model. The hydrogeological parameter values are chosen based on the results from the groundwater flow modelling and with the purpose that the chosen values as far as possible are valid for all repository parts during the whole time period.

#### 2.4 Base scenario - Biosphere

To calculate the distribution of radionuclides in the biosphere and the dose to human four different cases have been defined. The reasonable development of the biosphere is based on the expected development of the biosphere and future changes of the discharge area for the groundwater passing the repository. In addition cases with a mire area and a well are studied to illustrate uncertainties in discharge areas and biosphere development. To show the influence of the repository independent of changes in the biosphere one case is where today's biosphere is assumed to prevail during the whole time period. A more detailed description of the calculations of the migration of radionuclides in the different ecosystems and the different exposure pathways to human that are included in the calculations of dose is given in Karlsson *et al.*, (2001).

#### 2.4.1 Reasonable biosphere development

The reasonable case assumes that the release of radionuclides from the time for repository closure until 5000 AD is to a coast recipient that corresponds to a part of the present Öregrundsgrepen. In the calculations the same conditions as today is assumed to prevail until 4000 AD. The ongoing land rise will decrease the size of Öregrundsgrepen with time. For the time period 4000 AD to 5000 AD the water volume and water turn over is thus assumed to be less than today. At 5000 AD the radionuclides are released to a lake that is developed in the area. This lake will remain as recipient until 8000 AD. At that point the lake has grown into wet-land and mire areas has developed. After 8000 AD these areas are assumed to be drained and used as agricultural land until 12 000 AD. This reasonable biosphere development is schematically described in *Figure 2-2*.



Figure 2-2. Schematic description of the reasonable biosphere development.

Radionuclides released to the Öregrundsgrepen are assumed to be distributed between the water phase and suspended material in the water. The water turnover in Öregrundsgrepen imply that both dissolved radionuclides and radionuclides in suspended matter are spread over the whole water volume and further out in the Baltic Sea. The suspended matter with its content of radionuclides can settle as sediment and the nuclides can accumulate in deeper sediments but also move up to the water phase again. Vegetation in the water can take up radionuclides from the water while fish take up radionuclides both directly from the water and through contaminated food. Cattle that graze on the edge of the water can take up radionuclides by eating water vegetation and drink water. Humans are exposed for radionuclides by eating meat and drinking milk from cows that have been grazing at the shore and by eating fish from the area.

Radionuclides released to the lake can in the same way as in the coastal model be distributed between the water phase and suspended matter in the water. Dissolved radionuclides and radionuclides on suspended matter can also leave the system through discharge from the lake. Radionuclides on the suspended matter can settle on sediments and, if not resuspended to the water phase, become accumulated in deeper sediments as a consequence of burial of fresh sediment.

In the same way as in the coastal model radionuclides can be taken up by fish and water vegetation. Cattle grazing on the shores may take up radionuclides by eating water vegetation and drinking water from the lake. In contrast to the coastal model it is assumed in the lake model that humans take water from the lake both as drinking water and to irrigate a garden plot where vegetables and root crops are grown. Radionuclides in the irrigation water are thus directly transferred to the upper soil layer in the garden plot but also to some extent directly to the crop surfaces through the water droplets that are maintained on the leaves and other vegetation parts above the ground. Vegetables and root crops growing in the garden may, through root uptake, take up the radio-nuclides in the upper soil layer but also bring back radionuclides to the upper soil layers occur through processes like infiltration and bioturbation. Radionuclides in the deeper soil layers may also leak back to the lake and radionuclides in the upper soil layer may both return to the lake and leave the system through erosion.

Humans in the area can be exposed to radionuclides in the system by eating fish from the lake and by consumption of meat and milk from the cattle that grazes at the shores and drink water from the lake. In addition, exposure occurs through drinking water that is taken from the lake and through consumption of vegetables and root crops grown in the garden plot. Exposure via consumption of a small amount of soil, which is unintentionally eaten because of e.g. insufficiently washed vegetables, are also considered. Humans in the area can also be exposed by inhalation of contaminated soil particles in the air. In addition, it is assumed that working at the garden plot imply external exposure from the radionuclides in the soil.

The discharge area is changed to agricultural land at 8000 AD. The radionuclides are then supplied with the groundwater flowing in the saturated zone. These radionuclides can be transported up to the soil layer above through capillary forces, root uptake and through diffusion during dry periods while infiltration transports the radionuclide in the opposite direction during wet periods. Besides the radionuclides supplied with the groundwater the radionuclides accumulated in the sediments during the earlier coastal and lake periods are movable in the same way as the radionuclides in the groundwater. The concentration of radionuclides in the water phase is influenced by sorption on the solid materials. Radionuclides are transported out from the system with groundwater that leaves the system and due to erosion in the upper soil layer. Radionuclides in the upper soil layer are available for root uptake and may be transferred to crops cultivated in the area in form of cereals, root crops and vegetables. Part of the area is also used as pasturage for cattle. Humans are exposed to these radionuclides by eating the crops and by eating meat and drinking milk from cattle fed with crops and grass grown in the area. In addition exposure by unintentional intake of contaminated soil particles left on vegetables are also considered. Inhalation of contaminated soil particles as well as external exposure when working on the field are also included.

#### 2.4.2 Release to a mire area

This case is chosen to illustrate the uncertainty of discharge area for water and radionuclides and other uncertainties in future biosphere conditions. The reasonable biosphere development is based on a prognosis that shows that the discharge area will follow the shoreline. In this case it is instead assumed that the radionuclides are discharged into a mire area from year 4000 AD until 12 000 AD.

Radionuclides released into the mire area are distributed between the water phase and solid material. Out transport of radionuclides from the system occurs by water leaving the system from the layer close to the ground surface. It is assumed that peat from the mire area is used as soil improvement and as fuel in one household. The cultivated crops are taking up radionuclides from the peat and humans are exposed by consumption of the crops as well as meat and milk from cattle fed with the crops. In addition exposure is assumed to occur through inhalation of peat particles that are present in the air and through inhalation of the exhaust gas from peat combustion. External exposure from the mire area during outdoor staying in the area is also included.

#### 2.4.3 Release to a well

The well case illustrates the consequences of release of radionuclides to a well that some time in the future is sunk downstream the repository area. The results of the calculation of the future hydrogeology in the area indicates that such a well can not exist before 4000 AD. In the well case the consequences of release of radionuclides from the repository to a well downstream the repository during the time period 4000 AD to 12 000 AD are analysed.

Based on the hydrogeological calculations (Holmén and Stigsson, 2001a) it is assumed that all radionuclides released from one repository part exit in this well with a water consumption of 2,37 m<sup>3</sup>/day. Such a well can supply a small agricultural property with 5-10 cows and can also be used for irrigating a small garden plot where vegetables and root crops are grown. The turn-over of radionuclides that occurs by irrigation and the uptake through the food chains that this may cause is assumed to occur in the same way as in irrigation with lake water (see Section 2.4.1). One difference is that no nuclides may leave the system through erosion in the well case, so the well, crops and the upper and lower soil layers are in this respect a closed system.

Humans that take water from the well are supposed to be exposed by drinking water and by eating vegetables and root crops from the garden plot irrigated with water from the well. Uptake of radionuclides occurs also by consumption of meat and milk from cows that drink water from the well. Exposure also occurs by unintended intake of soil as well as through inhalation of contaminated soil particles in the air and through external exposure from radionuclides in the upper soil layer.

#### 2.4.4 Release to today's biosphere

The model used is the same as that for the first 2000 years in the reasonable biosphere development (see Section 3.4.1), i.e. a coastal model describing the conditions in Öregrundsgrepen today. The exposure pathways considered are thus consumption of fish as well as consumption of milk and meat from cattle, which has been grazing at the shores.

### 2.5 Other scenarios

#### 2.5.1 Initial defects

To illustrate the consequences of deviations from the expected properties of technical barriers and waste at repository closure three cases are studied:

- One or several large fractures in the concrete structure at repository closure
- Sorption in the technical barriers is considerably lower than in the base scenario due to considerable amounts of complexing agents that have been forgotten in the repository at closure
- The combined effect of a considerably lower sorption in the technical barriers and one or several large fractures in the concrete structure after 1000 years.

#### Fracture in the concrete structure

In this case it is assumed that, at closure of SFR 1, there are one or several large fractures in the concrete structures in BMA, 1BTF and 2BTF. These fractures are located in the vicinity of fracture zone 6 in the rock. Accordingly, this case is the same as the variation case in the base scenario with fractured/degraded barriers except for the point of time at which the fractures arise. Here it is assumed that fractures are already present at repository closure and the results from the hydrogeological modelling of the case with degraded barriers can therefore be used for the whole time period.

#### Chemicals/complexing agents

Two cases are studied to illustrate the effect of considerably deteriorated chemical barrier properties in the near field due to unexpected large amount of chemicals with complexing ability in for example the waste.

One case is identical to the main case in the base scenario except for that the sorption is considerably lower in all technical barriers in the Silo, BMA and BTF caverns for the elements influenced by complexing agents. The decreased sorption capacity is obtained by reducing the  $K_d$ -value in concrete/cement, bentonite, sand/bentonite and gravel backfill with a factor of 100 for tetravalent and pentavalent elements and with a factor of 10 for divalent and trivalent elements. In the other case, deteriorated sorption properties in the near field barriers described in the previous paragraph is combined with the variation case in the base scenario with degraded barriers after 1000 years.

#### 2.5.2 Climate - Permafrost

Even though it is very unlikely that continuous permafrost down to the depth of the repository will be formed before 12 000 AD, the effects of such a scenario is illustrated. For the sake of simplicity it is assumed that the whole repository is frozen until 12 000 AD when it melts. At this point all barriers are assumed to be cracked and not sustain any resistance for flowing groundwater. The chemical barrier function in form of sorption on the barrier materials does, however, remain. Radionuclides are transported out through the near field barriers with the total flow of groundwater in the

different repository parts. The size of the flow is obtained from the results of the calculation with the hydrogeological model on detail scale at 7000 AD, the end point of the calculations. The instant dose consequences at 12 000 AD are calculated for today's biosphere conditions.

#### 2.5.3 Human activity - Well in the repository

As intrusion scenario the consequences of a well sunk into the different repository parts are studied. A well can be sunk at 3000 AD at the earliest, when parts of the repository are not below the sea any longer. Here it is assumed that a well is sunk into the different parts of the repository where the largest water turnover prevail, i.e. the gravel backfill in the upper parts of the Silo, BMA and BTF caverns and in BLA. The water in the well is assumed to be used only as drinking water since drinking water is the dominating exposure pathway initially.

The long-term consequences of wells sunk into the repository are that an open hole constitutes a fast transport path for radionuclides from the repository up to the ground surface and that the water flow rate through the repository is influenced. If a well is sunk into BLA and this hole is left open, the water flow rate through BLA is higher than the flow obtained for the base case. An increase of a factor three at 3000 AD and a factor seven at 5000 AD is obtained (Holmén and Stigsson, 2001a).

## 3 Models

#### 3.1 Computer codes

#### 3.1.1 Near-field code, NUCFLOW

The near-field code NUCFLOW is a compartment model developed from the NUCTRAN-code (Romero, 1995 and Romero *et al.*, 1999). The major development in NUCFLOW is the possibility to use stepwise changes of the water flow with time.

NUCFLOW is a multiple path model that calculates the instationary nuclide transport in the near field of a repository as occurring through a network of resistances and capacitances coupled together in analogy with an electrical circuit network. The code takes into account diffusive and advective (water flow) transport, chain decay and sorption. It can simultaneously handle several sources, pathways and sinks to water flowing in fractures intersecting the repository.

To represent the barrier system, through which the species are transported, NUCFLOW makes use of the integrated finite difference method and of the concept of compartments. The barrier system is discretized into compartments. The material balance over a compartment connected to some other compartment for a dissolved single nuclide is described as:

$$V_i K_i \frac{dc_i}{dt} = \sum_{j \neq i} \left(\frac{AD_e}{d}\right)_{i,j} (c_j - c_i) + \sum_{j \neq i} \left(q_{j,i} c_j\right) - \sum_{j \neq i} \left(q_{i,j} c_i\right) - V_i K_i \lambda c_i$$

where

 $V_i$  is the volume of compartment i (m<sup>3</sup>)  $V_iK_i$  is the capacity of the compartment,  $K = \varepsilon + (1 - \varepsilon)K_d \rho_s$   $\varepsilon$  is the porosity of the material in the compartment (-)  $K_d$  is the distribution coefficient (m<sup>3</sup>/kg)  $\rho_s$  is the solid density (kg/m<sup>3</sup>)  $c_i, c_j$  is the concentration in compartment i and j respectively (mol/m<sup>3</sup>) t is time (s) A is the diffusion area (m<sup>2</sup>)  $D_e$  is the effective diffusivity (m<sup>2</sup>/s) d is the diffusion length (m) q is the water flow (m<sup>3</sup>/s)  $\lambda$  is the decay constant (s<sup>-1</sup>).

The left-hand side of the equation accounts for the accumulation of nuclides in the water and the solid by sorption. The right hand side accounts for the diffusive transport from one compartment to the adjacent compartments, the advective transport from one compartment to the adjacent compartments and the radionuclide decay.

The compartments are defined by their volume, their diffusion length and cross sectional area used by the diffusion and by their material data, such as porosity, density and diffusivity.

The equivalent water flow rate,  $Q_{eq}$ , is used for representing the diffusive transport of radionuclides from the compartments in contact with the far field (host rock). This fictitious flow rate can be visualised as the flow rate that carries away dissolved species

with the concentration at the compartment interface resulting in the release of radionuclides. It has been derived by solving equations for diffusive transport to the passing water by boundary layer theory. The value of  $Q_{eq}$  depends on the geometry of the contact area, the water flux, the flow porosity and the diffusivity as follows (Neretnieks *et al.*, 1987):

$$Q_{eq} = A_w \, \varepsilon_{rock} \, \sqrt{\frac{4 \, D_w}{\pi \, t_{res}}}$$

where

 $A_w$  is the surface area between the compartment and rock (m<sup>2</sup>)  $\varepsilon_{rock}$  is the flow porosity (-)  $D_w$  is the diffusivity of the nuclides in water (m<sup>2</sup>/s)

 $t_{res}$  is the residence time of the water in contact with the compartment (s).

#### 3.1.2 Far-field code, FARF31

The far-field code FARF31 used is the PROPER-version, Version 1.1.1. The PROPERpackage is a complete program package developed by SKB for calculation of the radionuclide release from a spent fuel repository.

The code FARF31 (Norman and Kjellbert, 1990 and Eriksson *et al.*, 1999) calculates the transport of dissolved radionuclides through the fractured rock, the retention caused by interactions between the nuclides and the rock matrix, and the radioactive chain decay. The processes included are:

*advection* - transport of radionuclides by water flowing through fractures in the rock *dispersion* - the spreading caused by velocity variations in different fractures or in different parts of a fracture

*matrix diffusion and sorption* - the diffusive transport of radionuclides from the water in the fracture into pores and microfissures of the rock matrix where the nuclides may sorb on the solid surfaces

*radioactive chain decay* - the decay and in-growth of radionuclides that are members of a decay chain.

FARF31 is based on the one-dimensional advection-dispersion equation with onedimensional diffusion perpendicular to the flow into a matrix of finite depth. The equation is formulated in flux averaged quantities of concentration, water velocity, dispersivity and the exchange rate between flowing water and the pores of the rock matrix.

#### 3.1.3 Biosphere model

The model system used is based on compartment models. Models are set up for each ecosystem studied, such as lake, agricultural land and coastal areas (Karlsson *et al.*, 2001). The concept of the models is a dynamic model for calculating the distribution of radionuclides between major physical components of the biosphere like soil, water and sediments, the result of which are used to predict biological uptake and radiation doses along multiple pathways. The turnover of radionuclides is described by first order differential equations, which are solved numerically with ACTIVI from the computer code BIOPATH (Bergström *et al.*, 1982 and 1995). The PRISM-system (Gardner et al, 1983) was used to generate randomly drawn values according to the statistical distribution, given for each parameter. The resulting transfers between different compartments of the models are all described by transfer coefficients or rate constants expressed as the turnover of elements per year.

The instationary phase of the important physical and geochemical processes such as erosion, sorption and radioactive decay are taken into account. Processes like bioaccumulation and root uptake are accounted for by steady state concentration ratios (bioconcentration factors).

The concentration of radionuclides in food stuff is calculated assuming equilibrium between plants/animals and their environment, an assumption that is justified by the time scale in question and that is usually used in this kind of modelling (e.g BIOMOVS, 1996, Davis et al., 1993). Uptake in aquatic biota is calculated using bioaccumulation factors whereas uptake in terrestrial vegetation is simulated using root-uptake factors and translocation factors (for radionuclides added to the vegetation surface). Specific transfer factors are used when calculating the transfer of radionuclides to milk and cattle meat. The internal exposure of humans is a combination of exposure from radionuclides that are eaten and those that are inhaled. Exposure from the former ones is calculated by combining the concentration in food stuff, consumption rates and nuclide specific dose coefficients for conversion from activity to dose (becquerel to sievert) for ingested radionuclides. By combining concentration in air with inhalation rates and nuclide specific dose coefficients for conversion from activity to dose for inhaled radionuclides exposure via inhalation is estimated. External exposure is estimated through consideration of exposure times, concentration of radionuclides in the air or on the ground and nuclide specific dose coefficients for conversion from concentration of radionuclides to dose (becquerel to sievert) for external exposure.

Probabilistic calculations have been performed for the biosphere. Thousand realisations have been performed for each case. The distribution functions used as input data are given in SKB (2001a). However, the results in this report are only presented as arithmetic mean values.

The total dose for each calculation case for the studied scenarios is calculated using TS01 and SUM41 within the PROPER-package.

#### 3.2 Model descriptions

The model used for modelling the migration of radionuclides in each repository part is described in this section. A schematic layout of the compartments is given in Appendix B and an example of an input data file for NUCFLOW for each repository part compartments is given in Appendix C. Schematic layouts of the biosphere model systems are given in Appendix D and the models are described in Karlsson *et al.*, (2001).

#### 3.2.1 Silo

A schematic figure of the model used for the radionuclide release calculations for the Silo is shown in *Figure 3-1*. The internal of the Silo has been divided into two sections in the radial direction and three in the vertical direction. The division in the radial direction is made to describe that the waste conditioned with bitumen is allocated to the inner part of the Silo surrounded by waste conditioned with cement. Approximately 20 % of the waste are embedded in bitumen, which means that the inner part with waste in the model is significantly smaller than the outer part. Three different waste types have been defined in the model; waste conditioned with cement in concrete moulds, waste conditioned with cement in steel packaging and waste conditioned with bitumen in steel packaging.

The division in three vertical sections is made to get a better representation of the transport. Each section is in turn divided into compartments representing different parts

of the repository, e.g. waste, concrete backfill and concrete walls. In total the model consists of 94 compartments. A detailed figure of the discretization is given in Appendix B.

Radionuclides in waste conditioned with cement in concrete moulds and in steel packaging are transported out from the waste package both by diffusion and advection. The diffusivity used for the waste matrix is that in concrete grout. Water flowing through the walls of the moulds is assumed to flow in small fractures in the concrete material and hence, sorption in mould walls is not taken into account for the advective transport through the mould walls. The diffusivity in the walls of the moulds is that in construction concrete. Radionuclides in waste conditioned with bitumen are assumed to be dissolved from bitumen to the water filled pores in the bitumen matrix at a constant rate during 100 years. As soon as the nuclides are in solution, they are free to be transported out by diffusion through the waste matrix to the surrounding concrete grout using the diffusivity in water. No sorption in bitumen is accounted for.

The main part of the concrete walls inside the Silo, forming the shafts, is only accounted for as sorption capacity that is available when nuclides diffuse into them. However, walls separating waste embedded in bitumen in the inner part of the Silo from cement conditioned waste in the outer part represent both a sorption capacity and a diffusive resistance.

For each section, the diffusion length in the concrete grout surrounding the waste packages is assigned a thickness of 14 cm for bitumenised waste and 20 cm for cement conditioned waste. It is assumed that the diffusion length is the same for radial and axial transport. The retention of nuclides in the concrete grout is on average underestimated, since the majority of radionuclides have to be transported several meters through the concrete grout.

The water flow rate through the waste part of the Silo is very low due to the low hydraulic conductivity, particularly in the concrete structure and surrounding bentonite barriers. Water flowing through the waste part is predominantly directed in the vertical direction (Holmén and Stigsson, 2001a), and is directed upward during the first thousand years from repository closure and thereafter downward. In the model, the advective transport in the horizontal plane through the waste part and surrounding concrete and bentonite barriers is neglected but the diffusive transport is accounted for. The boundary condition outside the bentonite is a so-called equivalent flow rate,  $Q_{eq}$ . In the bottom and the top of the Silo diffusion and advection in vertical as well as in horizontal direction is accounted for. There is a diffusive transport through the concrete lid of the Silo, but the water is assumed to flow in the gas vents only. Sorption is accounted for in all barriers within the Silo.



Figure 3-1. Transport paths for radionuclides in the Silo. View from side and above.

#### 3.2.2 BMA

A schematic figure of the model used for the radionuclide release calculations for BMA is shown in *Figure 3-2*. To describe the transport from BMA the repository is divided into five sections plus two parts representing gravel backfill in both ends of the vault. The division is made to get a better representation of the transport along the vault. The size of each section is chosen to comprise a certain number of rooms of the encapsulation. The first section represents room 1-4, the second represents room 5-9, the third represents room 10-11, the fourth represents room 12 and the fifth represents room 13-15. Each section is in turn divided into compartments representing different parts of the repository, e.g. waste, mould walls and encapsulation. In total the model consists of 161 compartments. A more detailed figure of the discretization is given in Appendix B.

In the model the waste have been sorted into five different waste types; concrete moulds with waste conditioned with cement, steel containers with waste conditioned with cement, steel drums with waste conditioned with bitumen and steel drums with waste conditioned with bitumen. It is foreseen that various metallic wastes will be deposited in BMA ("Other waste"). How this waste will be packaged is at present not specified, but in this study the waste is modelled as cement conditioned waste in steel containers. Each of the five sections of the model has a unique combination of the different waste types according to the present and planned allocation of the waste (Riggare and Johansson, 2001). This implies that the inventory of individual radionuclides is different in the five sections of the model.

In this study it is assumed that the space inside the encapsulation not occupied by waste is not backfilled. The high contrast in hydraulic conductivity between the space around the waste packages and the waste packages implies that the water flow through the waste packages will be limited. In the model it is assumed that no water is flowing through the packages. Radionuclides in waste conditioned with cement are transported out from the waste matrix by diffusion using the same diffusivity as in concrete grout. Radionuclides in concrete moulds also have to diffuse through the walls of the concrete moulds using a diffusivity of construction concrete. Radionuclides in waste conditioned with bitumen are assumed to be dissolved from bitumen to the water filled pores in the bitumen matrix at a constant rate during 100 years. As soon as the nuclides are in solution, they are free to be transported out by diffusion through the waste matrix using the diffusivity in water. Sorption is accounted for in waste conditioned with cement, but not for waste embedded in bitumen.





Figure 3-2. Transport paths for radionuclides in BMA. View from long side and short side.

#### 3.2.3 1BTF

In the vault 1BTF, concrete tanks are stacked two high around the walls of the vault, see *Figure 3-3*. Concrete moulds are used to make "compartments" where drums with ashes can be stacked. To make a separating "wall" the moulds are stacked four moulds high and nine moulds wide. A total of six compartments are needed to make room for 6 479 steel drums. Five of the compartments will be completely filled, containing 1 110 drums each, and the last compartment will hold 929 drums. When a compartment is filled with drums, it is sealed using concrete grout. The rest of the vault is used for concrete tanks, steel boxes and various metallic waste.

A schematic figure of the model used for the radionuclide release calculations for 1BTF is shown in *Figure 3-4*. To describe the transport from 1BTF the repository is divided into five sections plus two parts representing gravel backfill in both ends of the vault. The division is made to get a better representation of the transport along the vault. The first section contains steel drums with ashes, plus surrounding concrete tanks and concrete moulds. The concrete tanks and moulds are used as walls to facilitate the emplacement of the steel drums. However, in the model this is simplified and the three waste types are assumed to be in parallel with each other and surrounded by concrete grout. In the four remaining sections of 1BTF an equal distribution of concrete tanks and "Berglöfslöfslådor" has been assumed. Various metallic wastes foreseen to be deposited in 1BTF are modelled as concrete tanks. Each section is in turn divided into compartments representing different parts of the repository, e.g. waste, tank walls and gravel backfill. In total the model consists of 195 compartments. A more detailed figure of the discretization is given in Appendix B.

The main part of the water flows in the gravel in the top, and the part that flows through the waste and surrounding concrete grout is rather small. The contrast in hydraulic conductivity between the waste packages and the concrete grout is small, and water flow through the waste packages can not be disregarded. In the calculations the water flow that passes through the concrete grout is also assumed to pass through the waste packages.

The steel drums are 200 l drums in which a smaller drum with waste is deposited and the space between the two drums is filled with concrete. In the model, it is assumed that the whole volume between the two drums is filled with concrete. The sorption capacity in this concrete is used as the radionuclides diffuse and flow into the concrete. The concrete moulds used as internal walls for the drums are of the same type as in BMA, but only moulds with low activity are allocated to 1BTF. In the calculations activity in moulds in 1BTF is neglected, but the sorption capacity in the moulds is used as radionuclides from the drums diffuse and flow into them.



Figure 3-3. Schematic view of the allocation of waste in 1BTF.



Figure 3-4. Transport paths for radionuclides in 1BTF. View from long side and short side.

#### 3.2.4 2BTF

A schematic figure of the model used for the radionuclide release calculations for 2BTF is shown in *Figure 3-5*. To describe the transport from 2BTF the repository is divided into five sections plus two parts representing gravel backfill in both ends of the vault. The division is made to get a better representation of the transport along the vault. Since all the waste consists of ion exchange resins in concrete tanks, a homogeneous distribution of activity and waste packages is assumed. The model for 2BTF comprises

191 compartments totally. A more detailed figure of the discretization is given in Appendix B.

The water flows mainly in the gravel in the top of 2BTF and only a small part flow through the waste and surrounding concrete grout. The contrast in hydraulic conductivity between the waste packages and surrounding concrete grout is small and hence water flow through the waste packages may not be disregarded. In the calculations the water flow that passes through the concrete grout is also assumed to pass through the waste packages.



*Figure 3-5. Transport paths for radionuclides in 2BTF. View from long side and short side.* 

#### 3.2.5 BLA

A schematic figure of the model used for the radionuclide release calculations is shown in *Figure 3-6*. The model used for BLA is divided into five waste sections surrounded by a section with only water in each short side. The barriers in BLA are limited. In the model no barriers are taken into account. The nuclides are initially free to be transported with the water flow to the surrounding rock. The division into seven parts is probably of minor importance and the model may be seen as a stirred tank. This implies a high initial radionuclide release rate.



Figure 3-6. Transport paths for radionuclides in BLA. View from long side and short side.

## 4 Basic assumptions

This chapter summarises the premises on which this analysis is based upon. Also, the input data used for modelling the migration of radionuclides in the near field and the far field are compiled here. Motivations and a discussion of the selection of data is given elsewhere (SKB, 2001a).

#### 4.1 Premises

The radionuclide transport calculations are based on the following assumptions for the near field:

- The time it takes to refill the repository with groundwater after closure is neglected and all repository sections are saturated with water in year 2030.
- The construction of all technical barriers in the Silo and the vaults are in accordance with specifications for the existing facility and present plans for repository closure. For BMA it is assumed that the space between the waste packages is not backfilled.
- Waste packagings of steel are already fully permeable to water at repository closure.
- Radionuclides in bitumenised waste are released from bitumen to the water filled pores in the bitumen matrix with a constant rate during 100 years without being limited by solubility. The nuclide can diffuse to surrounding barriers as soon as it is in solution. Sorption on bitumen is not accounted for.
- All other radionuclides, i.e. unconditioned or conditioned/stabilised in cement, are assumed to be instantaneously dissolved in water in contact with the waste without being limited by solubility. The nuclides are transported to surrounding barriers through diffusion and, in some repository parts, by advection. Sorption is accounted for in cement conditioned or stabilised waste.
- The advective transport through the barriers is given by the magnitude and the direction of a water flow through them that is changing with time due to land rise. The results from the detailed hydrogeology modelling (Holmén and Stigsson, 2001a) have been adjusted to be in accordance with the discretization used in the migration model (Holmén and Stigsson, 2001b). The magnitude and direction of the water flow in three dimensions is then transferred as input data to the migration model. The water flow is changed stepwise at 3000 AD, 4000 AD and 5000 AD.
- Diffusion and sorption properties of the barriers are assumed to be constant in time. The conditions determining the sorption properties of the different barriers are the initial composition of the material, a high ion-strength in water and, for cement and concrete barriers, high pH.
- Waste packages in BLA are not surrounded by any barriers. The small amount of bitumen and concrete used for conditioning part of the waste in BLA is not accounted for. Neither is the bottom plate made of concrete or shotcrete covering walls and ceiling of vault.

- For the cases where the barriers are assumed to be intact, the hydraulic properties of all barriers are assumed to be constant in time. The hydraulic conductivity assigned to intact concrete barriers corresponds to concrete with small, intersecting fractures.
- For the cases where barriers are assumed to be degraded, the degraded barriers are given the same hydraulic properties as gravel.

#### 4.2 Near-field data

#### 4.2.1 Radionuclide inventory

The estimated inventory of radionuclides in SFR 1 at repository closure (Riggare and Johansson, 2001) includes 59 radionuclides, but only 42 of these are considered to be of importance for the safety assessment (SKB, 2001a). The initial inventory of these 42 nuclides is compiled in Table 4-1. Note that 14C has been divided into organic and inorganic carbon. However, in order to reduce the time needed for calculating the migration of radionuclides in near field and far field as well as estimating the corresponding doses, the number of nuclides included in the calculations has been limited to 29. This is discussed in more detail in Appendix A. As indicated in Table 4-1, <sup>241</sup>Pu is omitted in the calculations. However, the initial inventory (in mole) of <sup>241</sup>Pu is added to the initial inventory of the next nuclide in chain 4N+1, <sup>241</sup>Am. Chain decay is not taken into account with one exception, the formation of <sup>93m</sup>Nb from decay of <sup>93</sup>Mo.

Nuclide	Half-life	Silo	BMA	1BTF	2BTF	BLA
	[years] <sup>a)</sup>					
<sup>3</sup> Н	12.3	5.8·10 <sup>11</sup>	3.3·10 <sup>10</sup>	3.3·10 <sup>9</sup>	5.3·10 <sup>9</sup>	6.6·10 <sup>8</sup>
<sup>14</sup> C inorg	5730	2.0·10 <sup>13</sup>	1.9·10 <sup>12</sup>	2.3·10 <sup>12</sup>	2.7·10 <sup>11</sup>	3.9·10 <sup>10</sup>
<sup>14</sup> C org	5730	1.8·10 <sup>12</sup>	1.7·10 <sup>11</sup>	1.8·10 <sup>11</sup>	3.0·10 <sup>10</sup>	3.3·10 <sup>7</sup>
<sup>36</sup> Cl	301 000	4.7·10 <sup>10</sup>	3.4·10 <sup>9</sup>	3.0·10 <sup>8</sup>	5.4·10 <sup>8</sup>	8.2·10 <sup>7</sup>
<sup>60</sup> Co	5.27	1.8·10 <sup>15</sup>	7.1.10 <sup>13</sup>	5.4·10 <sup>12</sup>	9.1·10 <sup>12</sup>	1.0·10 <sup>12</sup>
<sup>59</sup> Ni	76 000	2.1.10 <sup>13</sup>	2.1.10 <sup>12</sup>	1.8·10 <sup>11</sup>	3.0·10 <sup>11</sup>	3.9·10 <sup>10</sup>
<sup>63</sup> Ni	100.1	3.6·10 <sup>15</sup>	3.2·10 <sup>14</sup>	2.9·10 <sup>13</sup>	4.7·10 <sup>13</sup>	6.2·10 <sup>12</sup>
<sup>79</sup> Se	1 130 000	1.9·10 <sup>10</sup>	1.4·10 <sup>9</sup>	1.2·10 <sup>8</sup>	2.2·10 <sup>8</sup>	3.3·10 <sup>7</sup>
<sup>90</sup> Sr	28.8	2.4·10 <sup>14</sup>	1.4·10 <sup>13</sup>	1.3·10 <sup>12</sup>	2.3·10 <sup>12</sup>	3.6·10 <sup>11</sup>
<sup>93</sup> Zr	1 530 000	2.1.10 <sup>10</sup>	2.1·10 <sup>9</sup>	1.8·10 <sup>8</sup>	3.0·10 <sup>8</sup>	3.9·10 <sup>7</sup>
<sup>93m</sup> Nb	16.1	7.6·10 <sup>12</sup>	4.9·10 <sup>11</sup>	4.7·10 <sup>10</sup>	7.6·10 <sup>10</sup>	9.7·10 <sup>9</sup>
<sup>94</sup> Nb	20 300	2.1·10 <sup>11</sup>	2.1·10 <sup>10</sup>	1.8·10 <sup>9</sup>	3.0·10 <sup>9</sup>	3.9·10 <sup>8</sup>
<sup>93</sup> Mo	4000	1.1.10 <sup>11</sup>	1.0·10 <sup>10</sup>	9.1·10 <sup>8</sup>	1.5·10 <sup>9</sup>	1.9·10 <sup>8</sup>
<sup>99</sup> Tc	211 000	2.4·10 <sup>13</sup>	1.7·10 <sup>12</sup>	1.5·10 <sup>11</sup>	2.7·10 <sup>11</sup>	4.1.10 <sup>10</sup>
<sup>107</sup> Pd	6 500 000	4.7·10 <sup>9</sup>	3.4·10 <sup>8</sup>	3.0·10 <sup>7</sup>	$5.4 \cdot 10^{7}$	8.2·10 <sup>6</sup>
<sup>108m</sup> Ag	418	1.2·10 <sup>12</sup>	1.2·10 <sup>11</sup>	1.0·10 <sup>10</sup>	1.7·10 <sup>10</sup>	2.2·10 <sup>9</sup>
<sup>113m</sup> Cd	14.1	8.2·10 <sup>11</sup>	3.6·10 <sup>10</sup>	3.6·10 <sup>9</sup>	6.4·10 <sup>9</sup>	1.0·10 <sup>9</sup>
<sup>126</sup> Sn	100 000	2.4·10 <sup>9</sup>	1.7·10 <sup>8</sup>	1.5·10 <sup>7</sup>	$2.7 \cdot 10^{7}$	4.1.10 <sup>6</sup>
<sup>129</sup>	$1.6 \cdot 10^{7}$	1.4·10 <sup>9</sup>	1.0·10 <sup>8</sup>	9.1.10 <sup>6</sup>	1.6·10 <sup>7</sup>	2.5·10 <sup>6</sup>
<sup>135</sup> Cs	2 300 000	2.4·10 <sup>10</sup>	1.7·10 <sup>9</sup>	1.5·10 <sup>8</sup>	2.7·10 <sup>8</sup>	4.1.10 <sup>7</sup>
<sup>137</sup> Cs	30.1	2.5·10 <sup>15</sup>	1.4·10 <sup>14</sup>	1.4·10 <sup>13</sup>	2.4·10 <sup>13</sup>	3.7·10 <sup>12</sup>
<sup>151</sup> Sm	90	1.1.10 <sup>13</sup>	7.5·10 <sup>11</sup>	6.9·10 <sup>10</sup>	1.2·10 <sup>11</sup>	1.9·10 <sup>10</sup>
<sup>152</sup> Eu	13.5	9.2·10 <sup>10</sup>	4.0·10 <sup>9</sup>	4.4·10 <sup>11</sup>	7.1·10 <sup>8</sup>	1.1.10 <sup>8</sup>
<sup>154</sup> Eu	8.59	7.9·10 <sup>13</sup>	2.7·10 <sup>12</sup>	2.8·10 <sup>11</sup>	4.8·10 <sup>11</sup>	7.6·10 <sup>10</sup>
<sup>166m</sup> Ho	1200	8.4·10 <sup>10</sup>	8.2·10 <sup>9</sup>	7.2·10 <sup>8</sup>	1.2·10 <sup>9</sup>	1.5·10 <sup>8</sup>
<sup>232</sup> U <sup>b)</sup>	68.9	2.0·10 <sup>7</sup>	1.1.10 <sup>6</sup>	8.1·10 <sup>4</sup>	6.3·10 <sup>4</sup>	6.1·10 <sup>4</sup>
<sup>234</sup> U <sup>b)</sup>	246 000	8.4·10 <sup>8</sup>	$4.5 \cdot 10^{7}$	3.5·10 <sup>6</sup>	2.7·10 <sup>6</sup>	2.5·10 <sup>6</sup>
<sup>235</sup> U <sup>b)</sup>	7.0·10 <sup>8</sup>	1.7·10 <sup>7</sup>	8.9·10 <sup>5</sup>	7.1·10 <sup>4</sup>	$5.4 \cdot 10^4$	5.1·10 <sup>4</sup>
<sup>236</sup> U <sup>b)</sup>	$2.3 \cdot 10^{7}$	2.5·10 <sup>8</sup>	$1.3 \cdot 10^{7}$	1.1.10 <sup>6</sup>	8.2·10 <sup>5</sup>	7.6·10 <sup>5</sup>
<sup>238</sup> U <sup>b)</sup>	4.5·10 <sup>9</sup>	3.3·10 <sup>8</sup>	$1.8 \cdot 10^{7}$	1.4·10 <sup>6</sup>	1.1.10 <sup>6</sup>	1.0·10 <sup>6</sup>
<sup>237</sup> Np <sup>b)</sup>	2 140 000	3.3·10 <sup>8</sup>	1.8·10 <sup>7</sup>	1.4·10 <sup>6</sup>	1.1.10 <sup>6</sup>	1.0·10 <sup>6</sup>
<sup>238</sup> Pu	87.7	2.8·10 <sup>12</sup>	1.5·10 <sup>11</sup>	1.1.10 <sup>10</sup>	8.9·10 <sup>9</sup>	8.5·10 <sup>9</sup>
<sup>239</sup> Pu	24 100	2.8·10 <sup>11</sup>	1.5·10 <sup>10</sup>	1.2·10 <sup>9</sup>	9.1·10 <sup>8</sup>	8.5·10 <sup>8</sup>
<sup>240</sup> Pu	6560	5.6·10 <sup>11</sup>	3.0·10 <sup>10</sup>	2.4·10 <sup>9</sup>	1.8·10 <sup>9</sup>	1.7·10 <sup>9</sup>
<sup>241</sup> Pu <sup>b)</sup>	14.4	3.1.10 <sup>13</sup>	1.6·10 <sup>12</sup>	1.0·10 <sup>11</sup>	8.2·10 <sup>10</sup>	8.8·10 <sup>10</sup>
<sup>242</sup> Pu	373 000	2.5·10 <sup>9</sup>	1.3·10 <sup>8</sup>	$1.1 \cdot 10^{7}$	8.2·10 <sup>6</sup>	7.6·10 <sup>6</sup>
<sup>241</sup> Am	432	6.1·10 <sup>12</sup>	4.3·10 <sup>10</sup>	3.4·10 <sup>9</sup>	2.6·10 <sup>9</sup>	2.5·10 <sup>9</sup>
<sup>242m</sup> Am <sup>b)</sup>	141	7.5·10 <sup>9</sup>	4.0·10 <sup>8</sup>	$3.1 \cdot 10^{7}$	$2.4 \cdot 10^{7}$	2.3·10 <sup>7</sup>
<sup>243</sup> Am <sup>b)</sup>	7370	2.5·10 <sup>10</sup>	1.3·10 <sup>9</sup>	1.1.10 <sup>8</sup>	8.1·10 <sup>7</sup>	7.6·10 <sup>7</sup>
<sup>243</sup> Cm <sup>b)</sup>	29.1	1.0.10 <sup>10</sup>	5.3.10 <sup>8</sup>	3.8·10 <sup>7</sup>	3.0.10 <sup>7</sup>	3.0.10 <sup>7</sup>
<sup>244</sup> Cm <sup>b)</sup>	18.1	1.1.10 <sup>12</sup>	5.9·10 <sup>10</sup>	3.9·10 <sup>9</sup>	3.1.10 <sup>9</sup>	3.3·10 <sup>9</sup>
<sup>245</sup> Cm <sup>b)</sup>	8500	2.5 10 <sup>8</sup>	1.3.10 <sup>7</sup>	1.1.10 <sup>6</sup>	8.1 · 10 <sup>5</sup>	7.6·10 <sup>5</sup>
<sup>246</sup> Cm <sup>b)</sup>	4730	6.7·10 <sup>7</sup>	3.6.10 <sup>6</sup>	2.8·10 <sup>5</sup>	2.2·10 <sup>5</sup>	2.0·10 <sup>5</sup>

Table 4-1. Radionuclide inventory (Bq) in SFR 1 at repository closure (2030) (Riggare and Johansson, 2001a).

a) Firestone (1998)

b) Nuclides not included in the final calculations.

#### 4.2.2 Physical and chemical data

Values on density, porosity and effective diffusivity for the materials modelled in SFR 1 are given in Table 4-2. Sorption coefficients used are compiled in Table 4-3 and Table 4-4. The latter table is valid for cases when the influence of chemicals, e.g. ISA, are accounted for. Note that the sorption coefficient is uninfluenced by chemicals for some elements. No sorption in bitumen has been accounted for.

Material	Solid density (kg/m³)	Effective diffusivity (m²/s)	Porosity (m³/m³)
Construction concrete	2529	1.10 <sup>-11</sup>	0.15
Concrete grout in Silo	2429	1·10 <sup>-10</sup>	0.30
Concrete grout in 1BTF/2BTF	2625	1·10 <sup>-10</sup>	0.20
Gravel and sand	2700	6·10 <sup>-10</sup>	0.30
Cement conditioning in packages	2250	1·10 <sup>-10</sup>	0.20
Bentonite	2692	1·10 <sup>-10</sup>	0.61
Sand/bentonite 90/10	2667	1·10 <sup>-10</sup>	0.25
Bitumen	1030	_	0
Water	1000	2·10 <sup>-9</sup>	1

Table 4-2. Physical data for materials used in SFR 1 (SKB, 2001a).

Table 4-3. Sorption	coefficients, K	<sub>d</sub> , (m³/kg) (SKB	, 2001a).
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Element	Concrete and cement	Gravel and sand	Bentonite	Sand/bentonite 90/10
Н	0	0	0	0
C (inorganic)	0.2	0.0005	0	0.0005
C (organic)	0	0	0	0
CI	0.006	0	0	0
Со	0.04	0.01	0.02	0.01
Ni	0.04	0.01	0.02	0.01
Se	0.006	0.0005	0	0.0005
Sr	0.001	0.0001	0.001	0.0002
Zr	0.5	0.5	0.05	0.5
Nb	0.5	0.5	0	0.5
Мо	0.006	0	0	0
Тс	0.5	0.3	0.01	0.3
Pd	0.04	0.001	0	0.0009
Ag	0.001	0.01	0	0.009
Cd	0.04	0.01	0.02	0.01
Sn	0.5	0	0.01	0.001
1	0.003	0	0	0
Cs	0.001	0.01	0.005	0.01
Sm	5	1	0.2	0.9
Eu	5	1	0.2	0.9
Ho	5	1	0.2	0.9
U	5	1	0.01	0.9
Np	5	1	0.1	0.9
Pu	5	1	1	1
Am	1	1	1	1
Cm	1	1	1	1

Element	Concrete and cement	Gravel and sand	Bentonite	Sand/bentonite 90/10
Н	0	0	0	0
C (inorganic)	0.2	0.0005	0	0.0005
C (organic)	0	0	0	0
CI	0.006	0	0	0
Co	0.004	0.001	0.002	0.001
Ni	0.004	0.001	0.002	0.001
Se	0.006	0.0005	0	0.0005
Sr	0.001	0.0001	0.001	0.0002
Zr	0.005	0.005	0.0005	0.005
Nb	0.005	0.005	0	0.005
Мо	0.006	0	0	0
Тс	0.005	0.003	0.0001	0.003
Pd	0.004	0.0001	0	0.00009
Ag	0.001	0.01	0	0.009
Cd	0.004	0.001	0.002	0.001
Sn	0.005	0	0.0001	0.00001
I	0.003	0	0	0
Cs	0.001	0.01	0.005	0.01
Sm	0.5	0.1	0.02	0.09
Eu	0.5	0.1	0.02	0.09
Ho	0.5	0.1	0.02	0.09
U	0.05	0.01	0.0001	0.009
Np	0.05	0.01	0.001	0.009
Pu	0.05	0.01	0.01	0.01
Am	0.1	0.1	0.1	0.1
Cm	0.1	0.1	0.1	0.1

Table 4-4. Sorption coefficients,  $K_d$  (m<sup>3</sup>/kg), influenced by chemicals (SKB, 2001a).

#### 4.2.3 Hydrological data

The water flow rate through the different repository parts of SFR 1 has been estimated and is discussed in detail elsewhere (Holmén and Stigsson, 2001a). The total water flow rate through the different repository parts of SFR 1 as estimated in the detailed hydrogeology modelling is given in Table 4-5. The results from the detailed hydrogeology modelling have been adjusted to be in accordance with the discretization used in the migration model (Holmén and Stigsson, 2001b). The adjusted data on magnitude and direction of water flow in three dimensions is then transferred as input data to the migration model. Accordingly, a three-dimensional water flow is assigned to each compartment in the migration model.

Table 4-5.	Total water flow rate through different repository parts	(Holmén and
Stigsson,	2001a).	

Repository part				
	2000 AD	3000 AD	4000 AD	5000 AD
BTF1: Waste part	2.4	2.7	6.8	7.8
BTF1: Whole vault	7.5	19.4	26.4	30.7
BTF2: Waste part	2.4	3.0	6.0	6.8
BTF2: Whole vault	6.7	17.6	27.7	29.6
BLA: Waste part	9.6	19.4	35.0	38.4
BLA: Whole vault	13.6	33.1	50.2	54.2
BMA: Encapsulation	0.07	0.13	0.26	0.28
BMA: Whole vault	8.7	36.7	52.7	54.7
Silo: Encapsulation	0.23	0.22	0.16	0.23
Silo: Top filling	0.53	1.4	2.2	2.2
Repository part	Total water flow rate (m <sup>3</sup> /yr)			
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	2000 AD	3000 AD	4000 AD	5000 AD
BTF1: Waste part				
Intact part	1.2	1.9	4.0	4.7
Degraded part	12.0	15.1	30.7	33.9
BTF1: Whole vault	14.5	24.1	45.0	50.2
BTF2	Same data assumed as for 1BTF			
BMA: Encapsulation				
Intact part	0.04	0.16	0.29	0.30
Degraded part	2.4	3.7	9.2	10.1
BMA: Whole vault	8.7	36.6	52.6	54.6
Silo: Encapsulation	0.66	1.1	1.5	1.5
Silo: Top filling	0.81	1.8	2.6	2.5

Tabell 4-6. Total water flow rate through different repository parts when these are degraded (Holmén and Stigsson, 2001a)

The equivalent water flow rate,  $Q_{eq}$ , used for representing the diffusive transport of radionuclides from the bentonite wall of the Silo to the far field (host rock) is given in Table 4-7.

Table 4-7. Equivalent water flow rate (m<sup>3</sup>/yr) for the Silo model.

	Q <sub>ea</sub> (m <sup>3</sup> /yr)			
Compartment*)	2000AD - 3000AD	3000AD - 4000AD	4000AD - 5000AD	5000AD –
40	0.004	0.023	0.026	0.028
41	0.019	0.021	0.024	0.026
42	0.020	0.004	0.004	0.005

\*) The Silo model with the compartments is given in Appendix B.

## 4.3 Far-field data

The data needed for modelling the far-field transport using FARF31 is summarised in Table 4-8. Radionuclide specific effective diffusivities,  $D_e$ , and sorption coefficients,  $K_d$ , used within this work are compiled in Table 4-9.

Table 4-8. Compilation of data used for modelling the migration of radionuclides in the far field (SKB, 2001a).

Parameter	
Water travel time, t <sub>w</sub> (yr)	50
Peclet number, Pe (-)	10
Flow-wetted surface area, $a_w (m^2/m^3)$	120
Matrix porosity, $\epsilon$ (m <sup>3</sup> /m <sup>3</sup> )	0.005
Maximum penetration depth, $x_0$ (m)	2

Table 4-9. Radionuclide specific effective diffusivites  $D_{\rm e}$  (m²/s) and sorption coefficients  $K_d$  (m³/kg) (SKB, 2001a).

Element	De	K <sub>d</sub>	
	Saline	Saline	
Н	1.0·10 <sup>-13</sup>	0	
Cinorganic	5.0·10 <sup>-14</sup>	0.001	
C <sub>organic</sub>	4.0·10 <sup>-14</sup>	0	
CI	8.3·10 <sup>-14</sup>	0	
Co	2.9·10 <sup>-14</sup>	0.02	
Ni	2.8·10 <sup>-14</sup>	0.02	
Se	$4.0 \cdot 10^{-14}$	0.001	
Sr	3.3·10 <sup>-14</sup>	0.0002	
Zr	4.0·10 <sup>-14</sup>	1	
Nb	4.0·10 <sup>-14</sup>	1	
Мо	$4.0 \cdot 10^{-14}$	0	
Tc(IV)	4.0·10 <sup>-14</sup>	1	
Pd	$4.0 \cdot 10^{-14}$	0.01	
Ag	7.1·10 <sup>-14</sup>	0.05	
Cd	3.0·10 <sup>-14</sup>	0.02	
Sn	$4.0 \cdot 10^{-14}$	0.001	
I	8.3·10 <sup>-14</sup>	0	
Cs	8.8·10 <sup>-14</sup>	0.05	
Sm	4.0·10 <sup>-14</sup>	2	
Eu	$4.0 \cdot 10^{-14}$	2	
Ho	4.0·10 <sup>-14</sup>	2	
U(IV)	4.0·10 <sup>-14</sup>	5	
Np(IV)	4.0·10 <sup>-14</sup>	5	
Pu	$4.0 \cdot 10^{-14}$	5	
Am	4.0·10 <sup>-14</sup>	3	
Cm	4.0·10 <sup>-14</sup>	3	

# 4.4 Biosphere data

The dose conversion factors for ingestion used for estimating the dose in the human intrusion scenario are summarised in Table 4-10. These factors are also used in the check of the screening of nuclides (Appendix A). The dose conversion factors for coast given in Table 4-10 are used for estimating the dose in the permafrost scenario. Other data used for modelling the transport of radionuclides in the biosphere and estimation of dose is presented and discussed in Karlsson *et al.* (2001) and is compiled in SKB (2001a).

Nuclide	Ingestion dose factor	Ecosystem specific dose conversion	
	[Sv/Bq]	factor for coast	
		[Sv/Bq]	
<sup>3</sup> Н	1.80·10 <sup>-11</sup>	2.68·10 <sup>-21</sup>	
<sup>14</sup> C inorg	5.80·10 <sup>-10</sup>	1.28·10 <sup>-18</sup>	
<sup>14</sup> C org	5.80·10 <sup>-10</sup>	1.28·10 <sup>-18</sup>	
<sup>36</sup> CI	9.30·10 <sup>-10</sup>	1.40·10 <sup>-19</sup>	
<sup>60</sup> Co	3.40·10 <sup>-9</sup>	5.86·10 <sup>-19</sup>	
<sup>59</sup> Ni	6.30·10 <sup>-11</sup>	1.72·10 <sup>-20</sup>	
<sup>63</sup> Ni	1.50·10 <sup>-10</sup>	4.09·10 <sup>-20</sup>	
<sup>79</sup> Se	2.90·10 <sup>-9</sup>	1.02·10 <sup>-17</sup>	
<sup>90</sup> Sr	2.80·10 <sup>-8</sup>	1.06·10 <sup>-18</sup>	
<sup>93</sup> Zr	1.10·10 <sup>-9</sup>	6.16·10 <sup>-20</sup>	
<sup>93m</sup> Nb	1.20·10 <sup>-10</sup>	1.05·10 <sup>-20</sup>	
<sup>94</sup> Nb	1.70·10 <sup>-9</sup>	1.51·10 <sup>-19</sup>	
<sup>93</sup> Mo	3.10 <sup>.</sup> 10 <sup>-9</sup>	1.00·10 <sup>-19</sup>	
<sup>99</sup> Tc	6.40·10 <sup>-10</sup>	1.22·10 <sup>-20</sup>	
<sup>107</sup> Pd	3.70·10 <sup>-11</sup>	9.27·10 <sup>-22</sup>	
<sup>108m</sup> Ag	2.30·10 <sup>-9</sup>	8.41·10 <sup>-19</sup>	
<sup>113m</sup> Cd	2.30·10 <sup>-8</sup>	5.79·10 <sup>-18</sup>	
<sup>126</sup> Sn	4.70·10 <sup>-9</sup>	5.92·10 <sup>-18</sup>	
<sup>129</sup>	1.10 <sup>.</sup> 10 <sup>-7</sup>	1.49·10 <sup>-17</sup>	
<sup>135</sup> Cs	2.00·10 <sup>-9</sup>	5.79·10 <sup>-19</sup>	
<sup>137</sup> Cs	1.30·10 <sup>-8</sup>	3.73·10 <sup>-18</sup>	
<sup>151</sup> Sm	9.80·10 <sup>-11</sup>	4.61·10 <sup>-21</sup>	
<sup>152</sup> Eu	1.40·10 <sup>-9</sup>	1.94·10 <sup>-19</sup>	
<sup>154</sup> Eu	2.00·10 <sup>-9</sup>	2.76·10 <sup>-19</sup>	
<sup>166m</sup> Ho	2.00·10 <sup>-9</sup>	9.50·10 <sup>-20</sup>	
<sup>238</sup> Pu	2.30·10 <sup>-7</sup>	3.92·10 <sup>-18</sup>	
<sup>239</sup> Pu	2.50·10 <sup>-7</sup>	4.28·10 <sup>-18</sup>	
<sup>240</sup> Pu	2.50·10 <sup>-7</sup>	4.27·10 <sup>-18</sup>	
<sup>242</sup> Pu	2.40·10 <sup>-7</sup>	4.10·10 <sup>-18</sup>	
<sup>241</sup> Am	2.00·10 <sup>-7</sup>	1.16·10 <sup>-17</sup>	

Table 4-10. Dose conversion factors for ingestion (EU, 1996) and ecosystem specific dose conversion factors for coast (Karlsson *et al.*, 2001).

# 5 Results - Base scenario

In this chapter, the results for the base scenario are presented. Results for other scenarios, e.g. the influence of uncertainty in sorption data, are presented in Chapter 6. A main case has been defined within the base scenario where the near-field barriers are intact, the retention in the far-field rock is neglected and the doses from release of radio-nuclides are based on a reasonable biosphere development. The effect of near-field barriers being degraded after 1000 years is also highlighted, as well as the importance of the far-field rock as a barrier. In addition, three variations of ecosystems have been investigated.

This chapter is divided into five sections, each section focusing on one repository part. All sections have the same disposition. First the release from the near field is discussed. Since the far field is not accounted for in the main case, the dose in recipient is based on the near-field release rate. The near-field release rate is therefore followed by the dose obtained for the reasonable biosphere development. Next the effect of an accounted far field is shown. Finally, the dose corresponding to a near-field release, neglecting the far field, to today's biosphere, mire and well is discussed.

The reasonable development of the biosphere in the area is described elsewhere (Kautsky, 2001). This has been the base for the *reasonable biosphere development case* used in this study. It assumes that the release of radionuclides from the time for closure of the repository to year 5000 AD occurs to a coastal recipient corresponding to a part of Öregrundsgrepen. It is assumed that the conditions of today prevail until 4000 AD. The size of Öregrundsgrepen will decrease as a consequence of the land rise and therefore a smaller water volume and longer water retention times has been assumed for the time period 4000 AD to 5000 AD. After 5000 AD the release of radionuclides occurs to a lake which has been developed in the area. This lake is used as a recipient until 8000 AD. At that time the lake has grown into a mire area. After 8000 AD it is assumed that this area is drained and used for agricultural purposes until 12 000 AD.

Two other recipients are also used in order to illustrate the uncertainties in where the groundwater from the repository will reach the biosphere after 4000 AD. The reasonable biosphere development case is based on the prediction that the release point will follow the shoreline. In the *mire recipient case* it is instead assumed that the radionuclides are released to a mire area during the time period 4000 AD to 12 000 AD. In the *well recipient case* the radionuclides are assumed to reach the biosphere within a well during the time period 4000 AD to 12 000 AD.

A forth case, the *today's biosphere case*, where the coastal recipient of today is assumed to prevail during the time period from the closure of the repository until 12 000 AD has also been used in accordance to the statements in the directions of SSI (SSI, 1998).

The results are presented in graphs where the release rate (Bq/yr) or the arithmetic mean dose (Sv/yr) is plotted as a function of time. As seen in for example *Figure 5-1*, there is a sudden increase, or sometimes a decrease, in near-field release rate, especially evident at 3000 AD. This is an artefact from the way a non-stationary water flow is taken into account in the near-field model (in the model a continuously changing water flow is represented by stepwise changes). This artefact propagates to the results on far-field release rate as well as dose in recipient. The figures on dose for the reasonable biosphere development shows a sudden increase in dose around 4000 AD and 5000 AD,

as well as an increase or decrease at 8000 AD. These stepwise changes in dose is due to the fact the recipient to which the nuclides are assumed to be released changes from one recipient to another, as described above.

In general the doses increase as the recipient in the reasonable biosphere development case changes from Öregrundsgrepen to a coast with smaller water volume as well as when the coast changes to lake. This is mainly due to decreased dilution as water volumes decrease and retention times increase. Another important factor is that additional exposure pathways are considered in the lake model compared to the coastal models as freshwater is also used for consumption and irrigation. The model used for the time period 8000 AD to 12 000 AD (agricultural land model) is very different from the other used as the inflow of radionuclides occurs within a porous medium (soil) instead of to a water volume. In this model accumulation of radionuclides in soil is an important process for the calculation of the concentration of nuclides in the soil profile whereas dilution is the most important process in the coastal and lake models.

In the agricultural land model it is also assumed that radionuclides which have accumulated in sediments earlier during the coastal and lake stages are present within the soil and available for exposure to humans. This can be seen as a peak in the dose curves at 8000 AD. Processes like erosion of the uppermost soil layer and mixing between soil layers results in a decrease of the accumulated radionuclides with time leading to decreasing doses. For some radionuclides it can be seen that the dose curves increase again after a certain decrease. This is because "new" nuclides are supplied with the groundwater. The lines in the dose curves are interpolated between a number of points and because relatively few time steps has been used in the calculations for this later time period the curves do not describe the decrease in dose fully properly. For nuclides, which sorb to solid matter in soil this decrease is slower than for more mobile nuclides but this can, unfortunately, not be seen in the figures.

One radionuclide which deserves special attention here is <sup>14</sup>C for which the dose curves decrease drastically when the use of the agricultural land model starts. This is due to that no root uptake of carbon is considered in the model (carbon is fixed by the green parts above the ground). Instead humans intake <sup>14</sup>C only through unintended consumption of soil (e.g. via insufficiently washed vegetables) and through consumption of milk and meat from cattle which have consumed contaminated soil when grazing. This leads to a much lower exposure for <sup>14</sup> Cin the agricultural land model compared to other nuclides, which are also taken up by crops. <sup>14</sup>C is also one of the nuclides which give the highest doses in the lake model (the uptake in fish is very large) which fortifies the impression of a drastically decrease in the dose curves at 8000 AD.

### 5.1 Silo

#### 5.1.1 Intact barriers

The modelling of the migration of radionuclides in the Silo gives a release from the near field that during the whole studied time period is dominated by organic <sup>14</sup>C (see *Figure 5-1*). In the beginning, <sup>3</sup>H gives the second highest release rate, but from about 2200 AD <sup>108m</sup>Ag is the second most dominating nuclide in terms of release rate. The maximum release rate of <sup>108m</sup>Ag is obtained at 3000 AD. The release of <sup>36</sup>Cl, <sup>93</sup>Mo and <sup>135</sup>Cs becomes more and more important from about 5000 AD. The release rate of these three nuclides is, however, some three orders of magnitude lower than for organic <sup>14</sup>C.

For a reasonable biosphere development the dose is dominated by organic  ${}^{14}C$  up to about 8000 AD (*Figure 5-2*), when the recipient is changed from a lake to agricultural land and a significantly lower dose is obtained from organic  ${}^{14}C$ . The dose is dominated

by <sup>129</sup>I, <sup>36</sup>Cl, <sup>79</sup>Se and <sup>93</sup>Mo instead. The dose during the agricultural period is less than  $10^{-8}$  Sv/yr. The maximum total dose obtained from release of radionuclides from the Silo is  $3 \cdot 10^{-6}$  Sv/yr which is at 5000 AD.

The far-field barrier (the geosphere) delays the release of radionuclides. How effective the far field rock is as a barrier varies from nuclide to nuclide. The effect of the far-field barrier on the release of radionuclides in the Silo is shown in *Figure 5-3*. By comparing *Figure 5-1* and *Figure 5-3* it is seen that it to a large extent is the same nuclides that dominate the release rate from both the near field and the far field. The difference in release rate between the near field and the far field is insignificant for organic <sup>14</sup>C, <sup>36</sup>Cl, <sup>93</sup>Mo and <sup>129</sup>I. These nuclides do not sorb in the geosphere. For nuclides like inorganic <sup>14</sup>C, <sup>59</sup>Ni and <sup>79</sup>Se a reduction in maximum release rate of about a factor two is obtained at most. For other nuclides the far-field rock is a more effective barrier. The maximum far-field release rate of <sup>90</sup>Sr and <sup>135</sup>Cs is between three and six times lower than that from the near field. The corresponding reductions in release rate of <sup>3</sup>H and <sup>108m</sup>Ag is up to two orders of magnitude and of <sup>137</sup>Cs four orders of magnitude.

When the nuclides are released to today's biosphere the whole studied period the dose obtained from release of radionuclides in Silo is very low (*Figure 5-4*). The maximum dose is less than  $10^{-10}$  Sv/yr and is dominated by the release of organic <sup>14</sup>C.

If the nuclides are released to a mire area after 4000 AD instead, higher doses are obtained (compare *Figure 5-5*). The dose dominating nuclide is organic <sup>14</sup>C between 4000 AD and 6000 AD and after that <sup>79</sup>Se. Other nuclides of importance for the total dose are <sup>108m</sup>Ag and <sup>36</sup>Cl. The maximum total dose  $(2 \cdot 10^{-7} \text{ Sv/yr})$  is obtained at the end of the near-field calculations (12 000 AD).

An even higher total dose is obtained if the nuclides are released to a well located downstream the repository (*Figure 5-6*). The maximum total dose  $(5 \cdot 10^{-5} \text{ Sv/yr})$  is obtained when the well is sunk (4000 AD), and is dominated by organic <sup>14</sup>C. The second most important nuclide is <sup>129</sup>I but its dose is between one and two orders of magnitude lower than that for the dominating nuclide.



*Figure 5-1.* Near-field release rate from Silo.(The nuclides in the bottom right corner are  $^{107}Pd$  and inorganic  $^{14}C$ .)



Figure 5-2. Dose from release of radionuclides from Silo to the reasonable biosphere.



Figure 5-3. Far-field release rate from Silo.



Figure 5-4. Dose from release of radionuclides from Silo to today's biosphere.



Figure 5-5. Dose from release of radionuclides from Silo to a mire area.



Figure 5-6. Dose from release of radionuclides from Silo to a well.

#### 5.1.2 Degraded barriers

Degradation of concrete and bentonite barriers leads to an increased water flow through the Silo. The magnitude and direction of the water flow has been estimated based on the results from the hydrogeology modelling with an increased hydraulic conductivity in the Silo (Holmén and Stigsson, 2001a). In the hydrogeology model a homogeneous conductivity, except for the gravel in the top, was assumed for this case. However, even though the barriers are degraded or fractured the hydraulic properties of the Silo should be such that an axial flow through the part of the Silo where the waste is located is dominating. A vertical flow through this part is therefore assumed in the migration model. The flow is directed upward during the first thousand years from repository closure and thereafter downward.

The results for a Silo being degraded 1000 years after repository closure are shown in *Figure 5-7* to *Figure 5-12*. Since the Silo is assumed to be intact during the first 1000 years, the water flow rate through the Silo, and thus the near-field release rate, during this time period are identical to that for the main case with intact barriers discussed in Section 5.1.1. The increase in flow 3000 AD leads to an increase in release rate compared to the main case. For <sup>59</sup>Ni between one and two orders of magnitude higher release rate from the near field is obtained, and for <sup>36</sup>Cl, <sup>79</sup>Se, <sup>93</sup>Mo, <sup>108m</sup>Ag, <sup>129</sup>I and <sup>135</sup>Cs the release rate is a factor seven to ten higher. The maximum near-field release rate of organic <sup>14</sup>C is a factor four higher than for the main case.

The delay in the far field may in this case with less retention in the near field be of more importance than in the case with intact barriers. However, the far field is of most importance for the short-lived nuclides and since this case is equal to the case with intact barriers the first 1000 years, the influence of the far -field is not markedly higher than in the case with intact barriers.

The maximum total dose for release to reasonable biosphere, today's recipient, mire area and well is  $9 \cdot 10^{-6}$  Sv/yr,  $3 \cdot 10^{-10}$  Sv/yr,  $5 \cdot 10^{-6}$  Sv/yr and  $2 \cdot 10^{-4}$  Sv/yr, respectively.



Figure 5-7. Near-field release rate from a Silo being degraded from 3000 AD.



*Figure 5-8.* Dose from release of radionuclides from a Silo being degraded from 3000 AD to the reasonable biosphere (calculations performed for dominating nuclides only).



Figure 5-9. Far-field release rate from a Silo being degraded from 3000 AD.



*Figure 5-10.* Dose from release of radionuclides from a Silo being degraded from 3000 AD to today's biosphere (calculations performed for dominating nuclides only).



*Figure 5-11.* Dose from release of radionuclides from a Silo being degraded from 3000 AD to a mire area (calculations performed for dominating nuclides only).



*Figure 5-12.* Dose from release of radionuclides from a Silo being degraded from 3000 AD to a well (calculations performed for dominating nuclides only).

## 5.2 BMA

#### 5.2.1 Intact barriers

The results for BMA for the base scenario are shown in *Figure 5-13* to *Figure 5-18*. The near-field release rate is dominated by organic <sup>14</sup>C, and from about 5800 AD and onwards by <sup>59</sup>Ni. The exception is the first few years after repository closure when <sup>90</sup>Sr results in a somewhat higher release rate than organic <sup>14</sup>C. <sup>90</sup>Sr gives a significant release rate for about 200 years. <sup>3</sup>H and <sup>137</sup>Cs are also among the nuclides dominating the near-field release shortly after repository closure.

Organic <sup>14</sup>C also dominates the total dose for the reasonable biosphere development until 8000 AD (see *Figure 5-14*). When the recipient changes from a lake to agricultural land at 8000 AD the dose corresponding to organic <sup>14</sup>C drops drastically but for some other nuclides it increases, and the total dose increases to  $2 \cdot 10^{-7}$  Sv/yr. During the agricultural period several nuclides contribute to the total dose, <sup>36</sup>Cl, <sup>59</sup>Ni, <sup>79</sup>Se, <sup>93</sup>Mo, <sup>135</sup>Cs and <sup>129</sup>I. <sup>79</sup>Se dominates the dose between 8000 AD and 9000 AD and later <sup>129</sup>I dominates. The maximum total dose for BMA is  $5 \cdot 10^{-7}$  Sv/yr, obtained at about 5000 AD.

The result on release from the far field is shown in *Figure 5-15*. The influence of the far-field rock on the release of radionuclides from BMA is a delay and thereby reduced release rates for especially sorbing short-lived nuclides. For example the release rate of <sup>90</sup>Sr, <sup>137</sup>Cs and <sup>63</sup>Ni decrease one or several orders of magnitude. The dose dominating nuclides are however only slightly influenced by the far field.

When the nuclides are released to today's biosphere the whole studied period the maximum total dose for release of radionuclides from BMA is very low (see *Figure 5-16*). A maximum dose of  $2 \cdot 10^{-10}$  Sv/yr is obtained and is dominated by the release of organic <sup>14</sup>C.

If the recipient is a mire area, the dose dominating nuclide is <sup>79</sup>Se. <sup>36</sup>Cl, <sup>59</sup>Ni and <sup>108m</sup>Ag each gives a dose that is about an order of magnitude lower. The dose from <sup>59</sup>Ni is still increasing at the end of the calculations. The total dose is almost constant during the studied time period with a maximum of  $3 \cdot 10^{-6}$  Sv/yr.

Release to a well results in an even higher total dose. The maximum dose is obtained at 4000 AD and is caused by the release of organic <sup>14</sup>C. Organic <sup>14</sup>C dominates the dose until 6000 AD. From then, <sup>129</sup>I and later <sup>59</sup>Ni are dominating nuclides. However, other nuclides, e.g. <sup>93</sup>Mo and <sup>135</sup>Cs, give a significant contribution to the total dose.



Figure 5-13. Near-field release rate from BMA.



Figure 5-14. Dose from release of radionuclides from BMA to the reasonable biosphere.



Figure 5-15. Far-field release rate from BMA.



Figure 5-16. Dose from release of radionuclides from BMA to today's biosphere.



Figure 5-17. Dose from release of radionuclides from BMA to a mire area.



Figure 5-18. Dose from release of radionuclides from BMA to a well.

#### 5.2.2 Degraded barriers

Degradation of the encapsulation in BMA, for example by leaching of components in the concrete structures, may lead to fracturing. It is foreseen that the largest effect on release from a degraded encapsulation is obtained where fracture zone 6 intersects BMA, since this is where the ground-water flow is highest. Fracture zone 6 intersects BMA at room 12 why it is assumed that this room is degraded. In this case it is assumed that degradation occurs instantaneously at 3000 AD (see Chapter 3). A fracture through this room have a limited influence on the total flow through BMA (Holmén and Stigsson, 2001a). A small redistribution of the flow is obtained, but the main effect is a significant increase in flow through the encapsulation at room 12.

The release rate from the near field for this case is shown in *Figure 5-19*. Since the flow through the repository between 2000 AD and 3000 AD is the same as for the main case shown in *Figure 5-13* above, the release rate for this period of time is identical. However, the higher water flow rate from 3000 AD compared to the main case generally results in an increased nuclide release rate. For certain time periods the release rate of some nuclides (e.g. inorganic <sup>14</sup>C, <sup>59</sup>Ni and <sup>99</sup>Tc) is between one and two orders of magnitude higher. Other nuclides, such as organic <sup>14</sup>C, <sup>36</sup>Cl, <sup>93</sup>Mo, <sup>129</sup>I and <sup>135</sup>Cs is only marginally affected by a higher water flow rate through room 12.

A significant difference between intact and degraded barriers in BMA is that the dose from release of <sup>59</sup>Ni, <sup>239</sup>Pu and <sup>240</sup>Pu to the reasonable biosphere after 8000 AD increases significantly with degraded barriers. This leads to that <sup>59</sup>Ni dominates until 10 500 AD and thereafter the two plutonium isotopes dominate the dose. The maximum total dose for release to reasonable biosphere in this case is  $1 \cdot 10^{-6}$  Sv/yr.

The release rate from the far field for this case is shown in *Figure 5-21*. Even though the retention in the near field barriers is smaller in this case compared to the case with intact barriers the influence of the far field is small for most of the dose dominating nuclides, except for <sup>239</sup>Pu and <sup>240</sup>Pu. The release of these nuclides is reduced more than two orders of magnitude.

The maximum total dose for release to today's biosphere, mire area and well is  $2 \cdot 10^{-10}$  Sv/yr,  $3 \cdot 10^{-6}$  Sv/yr and  $7 \cdot 10^{-5}$  Sv/yr, respectively.



*Figure 5-19. Near-field release rate from BMA with an encapsulation being degraded from 3000 AD.* 



*Figure 5-20.* Dose from release of radionuclides from BMA with an encapsulation being degraded from 3000 AD to the reasonable biosphere (calculations performed for dominating nuclides only).



*Figure 5-21. Far-field release rate from BMA with an encapsulation being degraded from 3000 AD.* 



*Figure 5-22.* Dose from release of radionuclides from BMA with an encapsulation being degraded from 3000 AD to today's biosphere (calculations performed for dominating nuclides only).



*Figure 5-23.* Dose from release of radionuclides from BMA with an encapsulation being degraded from 3000 AD to a mire area (calculations performed for dominating nuclides only).



*Figure 5-24.* Dose from release of radionuclides from BMA with an encapsulation being degraded from 3000 AD to a well (calculations performed for dominating nuclides only).

# 5.3 1BTF

### 5.3.1 Intact barriers

The results for the modelling of radionuclide transport within the near field of 1BTF are shown in *Figure 5-25*. The release is to a major part similar to that from BMA in terms of dominating nuclides. The maximum release rate for organic <sup>14</sup>C is  $2 \cdot 10^8$  Bq/yr which is obtained a thousand years after repository closure. From that point the release rate of organic <sup>14</sup>C decreases. Instead, the highest release rate is obtained by <sup>59</sup>Ni and inorganic <sup>14</sup>C from 6000 AD.

For release to the reasonable biosphere (*Figure 5-26*) the total dose is in the beginning dominated by organic <sup>14</sup>C. When the recipient for the nuclides is changed from a coastal area to a lake (at 5000 AD) there is a steep increase in dose. This increase is especially evident for inorganic <sup>14</sup>C, but organic <sup>14</sup>C is still resulting in the highest dose. The dose corresponding to inorganic <sup>14</sup>C is slowly increasing between 5000 AD and 8000 AD but that from organic <sup>14</sup>C is decreasing. The total dose is therefore dominated by inorganic <sup>14</sup>C between 6000 AD and 8000 AD. At 8000 AD the recipient is changed to an agricultural land, and consequently the dose from both organic and inorganic <sup>14</sup>C is significantly reduced. Several nuclides then significantly contribute to the dose, <sup>36</sup>Cl, <sup>59</sup>Ni, <sup>79</sup>Se, <sup>93</sup>Mo, <sup>129</sup>I and <sup>135</sup>Cs. At 8000 AD the dose is dominated by <sup>36</sup>Cl and later by <sup>129</sup>I. The maximum total dose, 4·10<sup>-7</sup> Sv/yr, is obtained at 5000 AD.

The influence of the far-field rock on the release of radionuclides from 1BTF is shown in *Figure 5-27*. The maximum far-field release rate of organic <sup>14</sup>C is a factor two lower than that from the near field. Other dose dominating nuclides, inorganic <sup>14</sup>C, <sup>36</sup>Cl, <sup>79</sup>Se and <sup>129</sup>I, are also slightly influenced by the far field. Short-lived sorbing nuclides are more influenced than the dose dominating nuclides. For example <sup>90</sup>Sr, <sup>137</sup>Cs, <sup>108m</sup>Ag and <sup>63</sup>Ni decrease one or several orders of magnitude in the far field.

The dose obtained from a release of radionuclides in 1BTF to today's biosphere is dominated by organic <sup>14</sup>C with a maximum total dose of  $3 \cdot 10^{-10}$  Sv/yr.

Release of radionuclides in 1BTF to a mire area gives a dose that is dominated by <sup>79</sup>Se.  $^{108m}$ Ag are among the dose dominating nuclides in the time period 4000 AD to 5000 AD (see *Figure 5-29*). Other nuclides of importance for the total dose are <sup>36</sup>Cl and <sup>59</sup>Ni, but they give a dose that is about an order of magnitude lower than <sup>79</sup>Se.

Organic <sup>14</sup>C dominates the dose between 4000 AD and 6000 AD when nuclides in 1BTF are released to a well (see *Figure 5-30*). For longer times several nuclides give a major contribution to the total dose, e.g. inorganic <sup>14</sup>C, <sup>59</sup>Ni, <sup>93</sup>Mo, <sup>129</sup>I and <sup>135</sup>Cs. The maximum total dose, obtained 4000 AD, is  $7 \cdot 10^{-5}$  Sv/yr.



Figure 5-25. Near-field release rate from 1BTF.



Figure 5-26. Dose from release of radionuclides from 1BTF to the reasonable biosphere.



Figure 5-27. Far-field release rate from 1BTF.



Figure 5-28. Dose from release of radionuclides from 1BTF to today's biosphere.



Figure 5-29. Dose from release of radionuclides from 1BTF to a mire area.



Figure 5-30. Dose from release of radionuclides from 1BTF to a well.

#### 5.3.2 Degraded barriers

The effect of degradation of barriers in 1BTF from 3000 AD has also been estimated. The water flow through 1BTF where a 17 m long section of the repository, located where fracture zone 6 intersects the vault, is given an insignificant flow resistance has been modelled (Holmén and Stigsson, 2001a). The results show that the water flow rate through the waste and surrounding concrete backfill is up to a factor six higher than for an intact barrier. Furthermore, it is concluded that the main part of the water in 1BTF (90 %) flows through the degraded part. However, the total water flow through the whole vault increases with no more than a factor two.

No radionuclide migration calculations have been made for this case. However, based on the results discussed in the previous paragraph, it is concluded that only waste packages in the degraded part will be affected by the higher flow rate obtained with a degraded barrier. Theoretically, in comparison to the nuclide release rate from an intact barrier, the release rate from these packages can increase with a factor corresponding to the increase in flow for the nuclides of importance. Radionuclides with short half-life may increase more. An increased release rate from the non-degraded part of the vault will not be obtained since the water flow rate through this part is not foreseen to increase. A 17 m wide section corresponds to about 10 % of the total waste volume. A flow increase of a factor six for 10 % of the waste together with essentially an uninfluenced water flow rate through the rest of the waste, and an increase in total flow through the vault of a factor two, both indicate that the near-field release rate should not be more than a factor two higher in comparison to the main case. Thus, the maximum total dose is estimated to be  $8 \cdot 10^{-7}$  Sv/yr for the reasonable biosphere development.

## 5.4 2BTF

### 5.4.1 Intact barriers

The results for the near-field release from 2BTF resembles that from BMA (compare *Figure 5-31* and *Figure 5-13*). In the beginning the release is dominated by organic <sup>14</sup>C and later by <sup>59</sup>Ni. The release rate of organic <sup>14</sup>C from 2BTF is about a factor five lower than that from BMA. The difference for <sup>59</sup>Ni is about a factor three.

The dose obtained for release of radionuclides from 2BTF for the reasonable biosphere development is shown in *Figure 5-32*. It is the same nuclides that dominate the dose as for BMA. The dose obtained from organic <sup>14</sup>C released from 2BTF is between a factor five to eight lower than the corresponding release from BMA. The maximum total dose is  $9 \cdot 10^{-8}$  Sv/yr at 5000 AD. When the lake is transferred into agricultural land at 8000 AD the total dose is  $5 \cdot 10^{-8}$  Sv/yr. <sup>79</sup>Se dominates the dose at this point of time followed by <sup>36</sup>Cl and <sup>59</sup>Ni. The dose decrease until 9000 AD, after that the dose increase slightly. During this time period <sup>36</sup>Cl, <sup>59</sup>Ni, <sup>79</sup>Se, <sup>93</sup>Mo, <sup>129</sup>I and <sup>135</sup>Cs dominate the dose.

The influence of the far-field rock on the release of radionuclides from 2BTF is shown in *Figure 5-33*. The dose dominating nuclides, organic <sup>14</sup>C, <sup>36</sup>Cl, <sup>59</sup>Ni, <sup>79</sup>Se and <sup>129</sup>I, are slightly influenced by the far field. Short-lived sorbing nuclides are more influenced than the dose dominating nuclides. For example <sup>90</sup>Sr, <sup>137</sup>Cs, <sup>108m</sup>Ag and <sup>63</sup>Ni decrease one or several orders of magnitude in the far field.

The total dose obtained when nuclides from 2BTF are released to today's biosphere is dominated by the release of organic <sup>14</sup>C, with a total dose less than  $10^{-10}$  Sv/yr (see *Figure 5-34*).

*Figure 5-35* shows the dose obtained for release to a mire area. The result for 2BTF is very similar to that obtained for BMA (*Figure 5-17*) in terms of dominating nuclides ( $^{36}$ Cl,  $^{59}$ Ni,  $^{79}$ Se and  $^{108m}$ Ag). The maximum total dose for 2BTF is however somewhat lower (7·10<sup>-7</sup> Sv/yr).

The results for 2BTF and BMA are also similar for a release to a well. Organic <sup>14</sup>C dominates the dose up to about 6000 AD and thereafter the dose is dominated by <sup>93</sup>Mo, <sup>129</sup>I and <sup>59</sup>Ni (see *Figure 5-36*). <sup>36</sup>Cl, <sup>79</sup>Se and <sup>135</sup>Cs also contribute to the total dose. The total dose from release from 2BTF is however somewhat lower than for BMA. The results for 2BTF gives a maximum total dose of  $1 \cdot 10^{-5}$  Sv/yr about 4000 AD.



Figure 5-31. Near-field release rate from 2BTF.



Figure 5-32. Dose from release of radionuclides from 2BTF to the reasonable biosphere.



Figure 5-33. Far-field release rate from 2BTF.



Figure 5-34. Dose from release of radionuclides from 2BTF to today's biosphere.



Figure 5-35. Dose from release of radionuclides from 2BTF to a mire area.



Figure 5-36. Dose from release of radionuclides from 2BTF to a well.

#### 5.4.2 Degraded barriers

Fracture zone 6 intersects approximately half way into 2BTF as for 1BTF and BMA. The development and the effect of such a degradation of 2BTF are therefore foreseen to be the same as for 1BTF. The maximum total dose is estimated to be  $2 \cdot 10^{-7}$  Sv/yr for the reasonable biosphere development.

### 5.5 BLA

Since no barriers have been accounted for in the migration model for BLA, the radionuclides can be transported momentarily from the waste to the surrounding rock. Consequently, a high release rate of radionuclides is obtained instantaneously (see *Figure 5-37*). The nuclides dominating the release a very short period of time after repository closure are shortlived nuclides like <sup>60</sup>Co, <sup>63</sup>Ni, <sup>90</sup>Sr and <sup>137</sup>Cs. The releases of most of these nuclides decrease rapidly and after 700 years the nuclides dominating the release are inorganic <sup>14</sup>C, <sup>59</sup>Ni and <sup>99</sup>Tc.

The dose for the reasonable biosphere development from radionuclide release from BLA is shown in *Figure 5-38* and *Figure 5-39*. The latter figure shows a detail of the dose during the first 1000 years. The release obtained directly after repository closure results in a total dose of approximately  $1 \cdot 10^{-8}$  Sv/yr, dominated by  $^{137}$ Cs. After about 300 years inorganic  $^{14}$ C is the dominating nuclide until about 7000 AD. From 7000 AD the dose is dominated by  $^{239}$ Pu and  $^{240}$ Pu. At 8000 AD the dose increase markedly and reaches its maximum,  $2 \cdot 10^{-7}$  Sv/yr.

The far field is a more important barrier for release of radionuclides from BLA than for release from the other repository parts. As shown in *Figure 5-37* there is a significant release of radionuclides in BLA from the near field from the very beginning, and all

nuclides reach its maximum release rate within about 1000 years from repository closure. When the nuclides are transported through the far-field rock the maximum release rate is reduced. How strong the effect is depends on the nuclide's half-life and its ability to sorb within the far-field rock. The maximum release rate of inorganic <sup>14</sup>C from the far field is less than a factor two lower than that from the near field. The corresponding factor is 3 for <sup>59</sup>Ni and 10 - 100 for <sup>63</sup>Ni, <sup>90</sup>Sr and <sup>99</sup>Tc. For <sup>239</sup>Pu and <sup>240</sup>Pu a reduction of three orders of magnitude is obtained, for <sup>137</sup>Cs four orders of magnitude and for <sup>60</sup>Co five orders of magnitude.

Transport through the far field also delays the release to the biosphere. Comparing the release curves from the near field (see *Figure 5-37*) and the far field (see *Figure 5-40*) for a specific nuclide this is evident. The far-field release of inorganic <sup>14</sup>C and <sup>59</sup>Ni resembles that from the near field to begin with and the time when the maximum release rate is obtained is roughly the same for the near field and the far field. However, when the maximum release rate has been obtained the reduction in far-field release rate with time is moderate in comparison to that from the near field. The far-field release rate of inorganic <sup>14</sup>C and <sup>59</sup>Ni therefore exceeds that from the near field as from 4000 AD. Other nuclides are transported more slowly through the far field. The release of <sup>99</sup>Tc, for example, is increasing with time and is of the same order of magnitude as for inorganic <sup>14</sup>C and <sup>59</sup>Ni around 8000 AD, with a maximum release rate obtained at12 000 AD. Several other nuclides, e.g. <sup>239</sup>Pu and <sup>240</sup>Pu, behave in the same way.

The far field is neglected in the base scenario. However, the extended release in time obtained from the far field in comparison to that from the near field of BLA, indicates that a higher dose could have been obtained if the dose was based on the release from the far field. The dose obtained from a far-field release to the reasonable biosphere has therefore been estimated for <sup>59</sup>Ni, <sup>99</sup>Tc, <sup>239</sup>Pu and <sup>240</sup>Pu. The obtained dose is shown in *Figure 5-41*.

The dose from the far-field release of  $^{59}$ Ni exceeds that obtained from the near-field release from 4000 AD. When nuclides are released to a lake (5000 AD – 8000 AD) the dose is between one and two orders of magnitude higher. When the recipient changes to agricultural land the difference in dose between near-field and far-field release is even larger, up to four orders of magnitude.

The K<sub>d</sub>-value for sorption of <sup>99</sup>Tc on the rock is higher than for <sup>59</sup>Ni. Accordingly, it takes longer time before the dose from the far-field release exceeds that from the near-field release (6000 AD). In the beginning of the agricultural period (8000 AD) the dose from the far-field and near-field release increase to the same maximum level about  $2 \cdot 10^{-9}$  Sv/yr. Thereafter the dose from the near-field release decrease markedly while the dose from the far-field release remains at almost the same level.

Plutonium has even better sorption characteristics. The dose obtained from the far-field release of <sup>239</sup>Pu and <sup>240</sup>Pu is therefore lower than that obtained from the near-field release.

BLA is the repository part giving the highest dose when the nuclides are released to today's biosphere (see *Figure 5-42*). The maximum total dose is about  $1 \cdot 10^{-8}$  Sv/yr. This dose is obtained directly after repository closure from release of <sup>137</sup>Cs. Besides the instant peak in dose, the maximum total dose is less than  $10^{-10}$  Sv/yr.

<sup>239</sup>Pu and <sup>240</sup>Pu are the two dominating nuclides in terms of dose when released from BLA to a mire area. The maximum total dose is  $3 \cdot 10^{-6}$  Sv/yr which is obtained 4500 AD (see *Figure 5-43*). Around 4000 AD <sup>99</sup>Tc is also one of the dominating nuclides but its

contribution is reduced rapidly with time. The third most important nuclide in terms of dose is <sup>79</sup>Se, but it gives a dose that is about one order of magnitude lower than the two plutonium isotopes.

When nuclides in BLA are released to a downstream well a dose exceeding  $10^{-4}$  Sv/yr are obtained instantaneously (maximum total dose is  $2 \cdot 10^{-4}$  Sv/yr). <sup>239</sup>Pu and <sup>240</sup>Pu dominate the dose for the whole time period studied (see *Figure 5-44*). Initially, <sup>99</sup>Tc gives an important contribution to the total dose as well.



Figure 5-37. Near-field release rate from BLA.



*Figure 5-38.* Dose from release of radionuclides from BLA to reasonable biosphere based on near-field release.



*Figure 5-39.* Dose from release of radionuclides from BLA to reasonable biosphere during the time period 2000 AD to 3000 AD based on near-field release.



*Figure 5-40. Far-field release rate from BLA.* 



*Figure 5-41.* Comparison of dose from release of radionuclides from BLA to reasonable biosphere. Based on release from a) near field (solid line) and b) far field (dashed line).



*Figure 5-42.* Dose from release of radionuclides from BLA to today's biosphere based on near-field release.



*Figure 5-43.* Dose from release of radionuclides from BLA to a mire area based on near-field release.



*Figure 5-44.* Dose from release of radionuclides from *BLA* to a well based on near-field release.
# 6 Results - Other scenarios

The results for scenarios not included in the base scenario are summarised in this chapter. Four scenarios are studied. These are the influence of:

- initially degraded barriers
- chemicals in waste
- permafrost
- human intrusion (drilling of a well into the repository).

### 6.1 Initially degraded barriers

In the main case within the base scenario it is assumed that the properties of the technical barriers fulfil the specifications. A case in which the barriers are degraded thousand years after repository closure is also investigated in the base scenario. But what happens if the properties of barriers and waste are not as good as assumed already at repository closure? A case in which there are one or several large fractures in the concrete structure in BMA already at repository closure has therefore been investigated. The fractures are formed close to fracture zone 6 that intersects the vault. It is the same scenario as the case with degraded barriers within the base scenario except for that there are fractures already at repository closure.

The main effect of an initial fracture through the concrete encapsulation in room 12 of BMA is a significant increase in water flow through that room and changed release paths from BMA. Initially the result is a decreased near-field release rate compared to the base case, but the higher water flow rate through room 12 causes the release to increase faster. For some nuclides it takes tens of years or less before the release rate with an initial fracture exceeds that for an intact structure. For other nuclides it takes hundreds of years. The increase in release rate during the first thousand years is a factor five or less. For organic <sup>14</sup>C for example the increase is a factor three at most. After 3000 AD the water flow is identical to the case in which the structure is assumed to be degraded after 1000 years from repository closure, see section 5.2.2. Between 3000 AD and 4000 AD small differences in nuclide release rate is obtained and for longer times the two cases are almost identical.

The total dose for release from an initially degraded BMA to the reasonable biosphere is estimated to be somewhat higher during the first thousand years after repository closure than for the case with degraded barriers after 1000 years (compare near-field release rate in *Figure 6-1* and *Figure 5-19*). Later only small differences will be obtained.

The effect of an initially degraded barrier in 1BTF or 2BTF on the maximum total dose for a reasonable biosphere will be the same as when the barrier is assumed to be degraded after 1000 years, see Section 5.3.2.

An initial fracture in the concrete bottom of the Silo does not affect the water flow rate through the Silo as long as the bentonite barriers are intact. The case with an initially degraded Silo has therefore not been studied.



Figure 6-1. Near-field release rate from BMA with an initially degraded room 12.



Figure 6-2. Far-field release rate from BMA with an initially degraded room 12.

# 6.2 Chemicals

The waste contains chemicals that may act as complexing agents for radionuclides. The waste also contains cellulose, which through alkaline degradation can form the strong complexing agent iso-saccarinic acid (ISA). Presence of complexing agents may reduce the nuclide's ability to sorb on different barrier materials within the repositories. In this study, this has been analysed by reducing the  $K_d$  value in concrete/cement, bentonite, sand/bentonite and gravel of trivalent, tetravalent and pentavalent ions (SKB, 2001a). All other data are identical to the main case within the base scenario. Since no sorption is accounted for in the migration model for BLA, no analysis has been made for this repository part.

The main part of the nuclides dominating the near-field release rate and/or the dose in reasonable biosphere for the main case in the base scenario for Silo, BMA, 1BTF and 2BTF are not affected by chemicals. The exception is <sup>59</sup>Ni, <sup>63</sup>Ni, <sup>94</sup>Nb, <sup>99</sup>Tc, <sup>126</sup>Sn and different plutonium isotopes. Other nuclides are also affected but they give a limited contribution to the total dose.

The release of radionuclides from Silo and BMA has also been calculated for a combined effect of chemicals and a degraded barrier after thousand years. No calculations have been made for 1BTF and 2BTF. Instead, based on the results for the case with chemicals and intact barriers and the knowledge of the effect of water flow through a degraded 1BTF and 2BTF, the effect of chemicals together with degraded barriers has been estimated.

The results for each repository part are presented graphically in the next sections. The release rate from near field and far field is calculated for all 30 modelled nuclides. However, nuclides foreseen to be insignificant in terms of contribution to the total dose in recipient are not included in the biosphere modelling.

### 6.2.1 Silo

### Intact barriers

The main difference for the near-field release between the main case (*Figure 5-1*) and this case is that the release rates of <sup>59</sup>Ni and <sup>99</sup>Tc approaches that of organic <sup>14</sup>C when influenced by chemicals, *Figure 6-3*. Also, more nuclides reach a release rate of between  $10^3$  and  $10^5$  Bq/yr.

For the reasonable biosphere development <sup>99</sup>Tc becomes the dominating nuclide from 8000 AD and onwards. However, the maximum total dose is the same as for the main case, dominated by organic <sup>14</sup>C that is not influenced by chemicals. The maximum total dose from release to today's biosphere is also the same dominated by organic <sup>14</sup>C. For release to mire area on the other hand, the dose corresponding to the release of <sup>59</sup>Ni and <sup>99</sup>Tc increase and is about the same as for <sup>79</sup>Se (not influenced by chemicals) (see *Figure 6-7*). The total dose from 6000 AD and onwards, and thus also the maximum total dose from release to a well is obtained initially at 4000 AD dominated by organic <sup>14</sup>C not influenced by chemicals. The total dose after that is increased by two to three times, mainly due to increase in dose from <sup>99</sup>Tc. <sup>99</sup>Tc becomes the dose dominating nuclide from about 11 000 AD.



Figure 6-3. Near-field release rate from an intact Silo affected by chemicals.



*Figure 6-4.* Dose from release of radionuclides from an intact Silo affected by chemicals to the reasonable biosphere (calculations performed for dominating nuclides only).



Figure 6-5. Far-field release rate from an intact Silo affected by chemicals.



*Figure 6-6.* Dose from release of radionuclides from an intact Silo affected by chemicals to today's biosphere (calculations performed for dominating nuclides only).



*Figure 6-7.* Dose from release of radionuclides from an intact Silo affected by chemicals to a mire area (calculations performed for dominating nuclides only).



*Figure 6-8.* Dose from release of radionuclides from an intact Silo affected by chemicals to a well (calculations performed for dominating nuclides only).

#### **Degraded barriers**

The release rates of many of the nuclides in the waste in the Silo, e.g.  $^{59}$ Ni and  $^{99}$ Tc, are affected both by chemicals and by degraded barriers. The near-field release rate of  $^{59}$ Ni and  $^{99}$ Tc for this case, shown in *Figure 6-9*, is about one order of magnitude higher than the case with only chemicals discussed previously in this section. Organic  $^{14}$ C dominating the total dose for the reasonable biosphere is, however, considered to be unaffected by chemicals.

The maximum total dose from release to the reasonable biosphere is about  $1 \cdot 10^{-5}$  Sv/yr at 5000 AD given by organic <sup>14</sup>C, identical to that obtained for the case with degraded barriers after 1000 years, see Section 5.1.2. After 8000 AD, the total dose is dominated by <sup>99</sup>Tc.

When the release occur to a mire area after 4000 AD, the maximum total dose is estimated to be  $7 \cdot 10^{-6}$  Sv/yr, predominantly given by <sup>79</sup>Se. If the nuclides are released to a well, the total dose is approximately constant in time, with a maximum dose of  $3 \cdot 10^{-4}$  Sv/yr. The dose is initially dominated by organic <sup>14</sup>C, followed by <sup>99</sup>Tc from about 6000 AD.

Dose in today's biosphere has not been estimated for this case, since the dose is expected to be low.



*Figure 6-9.* Near-field release rate from Silo affected by chemicals and being degraded from 3000 AD.



*Figure 6-10.* Dose from release of radionuclides from Silo affected by chemicals and being degraded from 3000 AD to the reasonable biosphere (calculations performed for dominating nuclides only).



*Figure 6-11.* Far-field release rate from Silo affected by chemicals and being degraded from 3000 AD.



*Figure 6-12.* Dose from release of radionuclides from Silo affected by chemicals and being degraded from 3000 AD to a mire area (calculations performed for dominating nuclides only).



*Figure 6-13.* Dose from release of radionuclides from Silo affected by chemicals and being degraded from 3000 AD to a well (calculations performed for dominating nuclides only).

### 6.2.2 BMA

#### Intact barriers

The release rate of <sup>59</sup>Ni and <sup>99</sup>Tc from the near field increases and exceeds that of organic <sup>14</sup>C from about 3400 AD and 3900 AD, respectively (see *Figure 6-14*). The release rate of these two nuclides  $(10^8 \text{ Bq/yr})$  is approximately constant in time from about 4000 AD.

The total dose for the reasonable biosphere development is determined by the release of organic <sup>14</sup>C up to 7000 AD and since <sup>14</sup>C is not influenced by chemicals the dose is unchanged. From 8000 AD and onwards the total dose increased about an order of magnitude when the effect of chemicals is accounted for in comparison to the main case within the base scenario (compare *Figure 6-15* and *Figure 5-14*).

The influence on dose in today's biosphere is negligible (compare *Figure 6-17* and *Figure 5-16*). If the recipient is a mire area the total dose will increase two to three times from 4000 AD and onwards. This is caused by the increased release of <sup>59</sup>Ni, <sup>99</sup>Tc, <sup>239</sup>Pu and <sup>240</sup>Pu (compare *Figure 6-18* and *Figure 5-17*). The dose in well is also higher when chemicals are accounted for. A total dose of about 0.2 - 0.3 mSv/yr is obtained between 4000 AD and 12 000 AD. <sup>99</sup>Tc is the dose dominating nuclide but <sup>59</sup>Ni, <sup>94</sup>Nb, <sup>239</sup>Pu and <sup>240</sup>Pu also give a dose exceeding the total dose for the main case within the base scenario from about 5000 AD.



Figure 6-14. Near-field release rate from an intact BMA affected by chemicals.



*Figure 6-15.* Dose from release of radionuclides from an intact BMA affected by chemicals to the reasonable biosphere (calculations performed for dominating nuclides only).



Figure 6-16. Far-field release rate from an intact BMA affected by chemicals.



*Figure 6-17.* Dose from release of radionuclides from an intact BMA affected by chemicals to today's biosphere (calculations performed for dominating nuclides only).



*Figure 6-18.* Dose from release of radionuclides from an intact BMA affected by chemicals to a mire area (calculations performed for dominating nuclides only).



*Figure 6-19.* Dose from release of radionuclides from an intact BMA affected by chemicals to a well (calculations performed for dominating nuclides only).

#### **Degraded barriers**

The combined effect of chemicals and degraded barriers has a smaller impact on the results for BMA than for the Silo. Comparing the near-field release rate for this case (*Figure 6-20*) with the case affected by chemicals (*Figure 6-14*) it is seen that there is a small increase in the release rate of <sup>59</sup>Ni and <sup>99</sup>Tc between 3000 AD and 4000 AD for the combined effect. From 5000 AD, however, the release rate is approximately the same for the two cases. For other nuclides, e.g. <sup>166m</sup>Ho, <sup>239</sup>Pu, <sup>240</sup>Pu and <sup>241</sup>Am, there is a larger difference. The release rates of <sup>239</sup>Pu and <sup>240</sup>Pu are at most one order of magnitude higher for the case with the combined effect. This is obtained around 5000 AD. The difference decreases with time and around 12 000 AD the release rate of <sup>239</sup>Pu and <sup>240</sup>Pu are more or less the same for the two cases. As was stated in Chapter 5, the near-field release rate of organic <sup>14</sup>C for the combined effect of chemicals. Accordingly, the near-field release rate of organic <sup>14</sup>C for the combined effect of chemicals and degraded barriers is identical to that for degraded barriers (see Section 5.2.2).

The total dose for release of radionuclides from BMA to the reasonable biosphere will initially to a large extent be the same as for the case with chemicals and intact barriers discussed in the previous section. The total dose up to 8000 AD is equivalent to that obtained for degraded barriers, see Section 5.2.2, since the dominating nuclides, organic <sup>14</sup>C and inorganic <sup>14</sup>C, are not affected by chemicals. For longer times, after 8000 AD, <sup>239</sup>Pu and <sup>240</sup>Pu result in the highest dose. An increase in total dose from 8000 AD and onwards of less than a factor two in comparison to that shown in *Figure 6-15* is obtained.

If the nuclides are released to a mire area instead, the dose corresponding to <sup>79</sup>Se, <sup>239</sup>Pu and <sup>240</sup>Pu will predominantly dominate the dose. Dose from release to a well is dominated by <sup>99</sup>Tc, <sup>239</sup>Pu and <sup>240</sup>Pu. The maximum total dose for release to mire area is about  $1 \cdot 10^{-5}$  Sv/yr and for release to a well about  $9 \cdot 10^{-4}$  Sv/yr.

Dose in today's biosphere has not been estimated for this case. The dose is expected to be low based on the results from the case with increased water flow rate through BMA discussed in Chapter 5 and the effect of chemicals in an intact BMA discussed above.



*Figure 6-20.* Near-field release rate from BMA affected by chemicals and being degraded from 3000 AD.



*Figure 6-21.* Dose from release of radionuclides from BMA affected by chemicals and being degraded from 3000 AD to the reasonable biosphere (calculations performed for dominating nuclides only).



*Figure 6-22.* Far-field release rate from BMA affected by chemicals and being degraded from 3000 AD.



*Figure 6-23.* Dose from release of radionuclides from BMA affected by chemicals and being degraded from 3000 AD to a mire area (calculations performed for dominating nuclides only).



*Figure 6-24.* Dose from release of radionuclides from BMA affected by chemicals and being degraded from 3000 AD to a well (calculations performed for dominating nuclides only).

#### 6.2.3 1BTF

#### Intact barriers

The results for the near-field release rate of nuclides affected by chemicals in 1BTF are given in *Figure 6-25*. The release rate of <sup>59</sup>Ni and <sup>99</sup>Tc exceeds that of organic <sup>14</sup>C from about 4800 AD. The release rate of these two nuclides is about one order of magnitude lower than from BMA.

The main effect of chemicals on the dose is caused by the increased release of <sup>59</sup>Ni, <sup>99</sup>Tc, <sup>239</sup>Pu and <sup>240</sup>Pu. This causes the maximum total dose for release to a mire area to increase by a factor two in comparison to the main case within the base scenario discussed in Section 5.3.1, even though <sup>79</sup>Se still dominates the total dose (see *Figure 6-29*). The maximum dose for release to the reasonable biosphere (*Figure 6-26*) or a well (*Figure 6-30*) increases marginally in comparison to the main case, to today's biosphere (*Figure 6-28*) not at all. However, in contradiction to the main case, the total dose in the well is almost constant from about 4000 AD to 12 000 AD when the effect of chemicals is accounted for (see *Figure 6-30*).



Figure 6-25. Near-field release rate from an intact 1BTF affected by chemicals.



*Figure 6-26.* Dose from release of radionuclides from an intact 1BTF affected by chemicals to the reasonable biosphere (calculations performed for dominating nuclides only).



Figure 6-27. Far-field release rate from an intact 1BTF affected by chemicals.



*Figure 6-28.* Dose from release of radionuclides from an intact 1BTF affected by chemicals to today's biosphere (calculations performed for dominating nuclides only).



*Figure 6-29.* Dose from release of radionuclides from an intact 1BTF affected by chemicals to a mire area (calculations performed for dominating nuclides only).



*Figure 6-30.* Dose from release of radionuclides from an intact 1BTF affected by chemicals to a well (calculations performed for dominating nuclides only).

#### **Degraded barriers**

In Section 5.3.2 it was stated that the water flow through 1BTF with part of the vault being degraded locally increases up to a factor six. However, the total water flow through the whole vault increases with no more than a factor two in comparison to an intact barrier. In accordance with the discussion in Section 5.3.2, a combined effect of chemicals and degraded barriers in 1BTF could give a total dose for the reasonable biosphere that is not more than a factor two higher than that for the case with chemicals and intact barriers. The total dose is expected to be dominated by organic and inorganic <sup>14</sup>C until 8000 AD and from that by <sup>99</sup>Tc.

#### 6.2.4 2BTF

#### **Intact barriers**

The near-field release rate for nuclides affected by chemicals from an intact 2BTF is shown in *Figure 6-31*. <sup>59</sup>Ni and <sup>99</sup>Tc gives the highest near-field release rate from about 3500 AD.

The effect of chemicals in 2BTF on dose in recipients is to a large extent the same as for 1BTF discussed in Section 6.2.3. The increase in dose in the reasonable biosphere development is however larger than for 1BTF. The maximum dose is in this case mainly achieved from <sup>99</sup>Tc at 8000 AD and is  $4 \cdot 10^{-7}$  Sv/yr.



Figure 6-31. Near-field release rate from an intact 2BTF affected by chemicals.



*Figure 6-32.* Dose from release of radionuclides from an intact 2BTF affected by chemicals to the reasonable biosphere (calculations performed for dominating nuclides only).



Figure 6-33. Far-field release rate from an intact 2BTF affected by chemicals.



*Figure 6-34.* Dose from release of radionuclides from an intact 2BTF affected by chemicals to today's biosphere (calculations performed for dominating nuclides only).



*Figure 6-35.* Dose from release of radionuclides from an intact 2BTF affected by chemicals to a mire area (calculations performed for dominating nuclides only).



*Figure 6-36.* Dose from release of radionuclides from an intact 2BTF affected by chemicals to a well (calculations performed for dominating nuclides only).

### **Degraded barriers**

The results for chemicals and degraded barriers in 2BTF are to a large extent foreseen to be the same as for the same case for 1BTF. However, the total dose for the reasonable biosphere development is expected to be dominated by organic <sup>14</sup>C for a shorter period of time, but will be followed by <sup>99</sup>Tc for 2BTF as well. The maximum dose will probably be determined by <sup>99</sup>Tc at 8000 AD as in the case with chemicals and intact barriers.

# 6.3 Permafrost

The climate is foreseen to be colder in the future, and thus permafrost could be obtained in the region. It has been estimated that this could happen earliest in 7000 AD (SKB, 2001b). To get a rough estimate of the consequence of permafrost, it is assumed that the repository is a frozen system from repository closure until 12 000 AD when the permafrost period is assumed to have ended. No radionuclides have been released before 12 000 AD, but radioactive decay for 10 000 years has caused the initial inventory to be reduced. All materials within the repository are mechanically degraded. The water flow is therefore homogeneously distributed within each repository part, and the radionuclides are transported out with the total flow in the repository part. The magnitude of the water flow rate is chosen to be the highest flow obtained within the hydrogeology modelling of SFR 1 (Holmén and Stigsson, 2001a). Sorption has been accounted for all barrier materials that are included in the near-field models.

The release is calculated as an instant release at 12 000 AD and the dose is calculated for the recipient that is representative for today's biosphere conditions. The result in terms of total dose from each repository part is shown in *Figure 6-37*. Organic <sup>14</sup>C dominates the dose for the Silo, BMA, 1BTF and 2BTF. For BLA the dose dominating nuclides are inorganic <sup>14</sup>C, <sup>239</sup>Pu and <sup>240</sup>Pu. Since there is a significant dilution in the recipient (Öregrundsgrepen), the dose is very low. The total dose for SFR 1 is  $1\cdot10^{-9}$  Sv/yr.



Figure 6-37. Total dose at 12 000 AD for permafrost scenario.

## 6.4 Human intrusion

A human intrusion scenario has been analysed in which a well is sunk directly into the gravel backfill in each repository part. Long-term effects have not been considered. Instead, the dose to man obtained at the time when the well is sunk is evaluated. This scenario can not be realised until the coastline has past the repository, i.e. from 3000 AD. Here, the scenario has been analysed assuming the well is sunk at 3000 AD, 4000 AD or 5000 AD.

In the calculations it is assumed for each repository part that the nuclide concentration in the well water is the same as the highest concentration in any part of the gravel backfill when the well is sunk. The concentrations are obtained from the migration calculations for the main case within the base scenario. The nuclide concentrations has been transformed into a dose using ingestion dose conversion factors (EU, 1996) and assuming a water consumption rate of 600 l/yr.

The total dose for each repository part for the defined intrusion scenario is shown in *Figure 6-38*. The highest total dose is obtained from a well in BLA if the well is sunk at 3000 AD. If the well is sunk at 4000 AD or 5000 AD on the other hand, a well in the Silo will result in the highest total dose. The dose dominating nuclide in BLA is  $^{240}$ Pu whereas  $^{14}$ C dominate in the other repositories.



Figure 6-38. Total dose for the analysed human intrusion scenario.

# 7 Summary of results

The consequences of migration of radionuclides from SFR 1 have been analysed for a base scenario. The base scenario describes the expected evolution of the repository's near field. Three different cases are defined within the base scenario. Intact barriers (main case) and degraded barriers after 1000 years are discussed in this report. The third case, gas influenced nuclide transport, is analysed in Moreno *et al.* (2001). Uncertainties in the biosphere development are taken into account by evaluating the consequences of releases to a reasonable biosphere, to today's biosphere, to a mire area and to a well.

Besides the base scenario, the effect of initially degraded barriers and the effect of the presence of materials in the waste that drastically deteriorate the possibility for nuclides to sorb on the barriers are analysed. In addition, a permafrost scenario and a human intrusion scenario are analysed.

In this last chapter, a conclusive summary of the results from the modelling of migration of radionuclides and corresponding doses in the biosphere is made. The main part of this chapter is devoted to the results for the base scenario, and then mainly the main case. Other scenarios are discussed more briefly. The maximum dose for all investigated cases is summarised in Table 7-1.

### 7.1 Base scenario

### 7.1.1 Intact barriers

The total dose from release of radionuclides from an intact SFR 1 with a reasonable biosphere development is shown in *Figure 7-1*. The total dose is initially dominated by the release of <sup>137</sup>Cs from BLA. From about 2250 AD to 4000 AD the dose is dominated by the release of organic <sup>14</sup>C from 1BTF and BMA. Between 4000 AD and 8000 AD organic <sup>14</sup>C from the Silo dominates the dose. The dominating pathway for exposure from <sup>137</sup>Cs as well as from organic <sup>14</sup>C is, during this whole time period, consumption of fish. The maximum total dose from SFR 1,  $4 \cdot 10^{-6}$  Sv/yr, is obtained when the recipient is changed from coast to a lake at 5000 AD. The dose from organic <sup>14</sup>C is reduced significantly when the recipient is changed from a lake to agricultural land at 8000 AD. Instead, <sup>239</sup>Pu and <sup>240</sup>Pu in BLA dominate the dose. The reason for the drastically decrease in dose from organic <sup>14</sup>C is the fact that carbon is not taken up in vegetation by roots. The exposure in the agricultural land model (which for most radionuclides is a product of root uptake) is therefore much lower. The uptake of carbon in fish is very large in the lake model comparing to most other nuclides making this contrast even larger. The most important pathways for exposure from <sup>239</sup>Pu and <sup>240</sup>Pu is inhalation of contaminated dust.



Figure 7-1. Total dose from release of radionuclides from SFR 1 to the reasonable biosphere.

An alternative to the reasonable biosphere development (coast, lake and agricultural land) is the development of mire. A mire can be formed at 4000 AD earliest (Kautsky, 2001). The total dose from release to a mire area from an intact SFR 1 is shown in *Figure 7-2*. Initially the total dose is dominated by the release of  $^{239}$ Pu and  $^{240}$ Pu from BLA, but the dose from radionuclides from BMA increases with time and exceeds that for BLA from 5400 AD. The dose from radionuclides in BMA is completely dominated by the release of  $^{79}$ Se. The total dose is more or less constant in time with a maximum of  $6 \cdot 10^{-6}$  Sv/yr around 4700 AD. The dose gained from exposure from  $^{239}$ Pu and  $^{240}$ Pu is due to inhalation of contaminated dust from the mire area. In contrast, the exposure from  $^{79}$ Se is to a large extent retained on the solid phase of the peat. This in combination with a large uptake by vegetation makes the potential of exposure large compared to other nuclides when a mire is chosen as a recipient.



Figure 7-2. Total dose from release of radionuclides from SFR 1 to a mire area.

Release to a well has also been analysed. It is assumed that the well has a capacity of  $2.37 \text{ m}^3$ /day and that it can be sunk no earlier than 4000 AD. Furthermore, the hydrogeological calculations show that the well may be placed so that either the release from all vaults or the release from the Silo will end up in the well (Holmén and Stigsson, 2001a).

The consequence of a release from an intact SFR 1 to a well is shown in *Figure 7-3*. <sup>239</sup>Pu and <sup>240</sup>Pu in BLA dominates the total dose for release from the vaults and organic <sup>14</sup>C dominate the release from the Silo. The maximum total dose for release from the vaults is  $4 \cdot 10^{-4}$  Sv/yr and from the Silo  $5 \cdot 10^{-5}$  Sv/yr. The dominating pathway for exposure from <sup>239</sup>Pu and <sup>240</sup>Pu is inhalation of contaminated dust from a garden plot, which is irrigated with the well water. For exposure from organic <sup>14</sup>C consumption of water is the most important exposure pathway followed by consumption of root crops and milk.



Figure 7-3. Total dose from release of radionuclides from SFR 1 to a well.

Release to today's biosphere results in a maximum dose of  $1 \cdot 10^{-8}$  Sv/yr obtained shortly after repository closure. With the exception of this instant release the maximum release rate is less than  $6 \cdot 10^{-10}$  Sv/yr.

### 7.1.2 Degraded barriers

Degraded barriers lead to a higher flow rate of water through the repository, and thus an increased release rate for some nuclides. The effect of a degraded barrier in Silo and BMA 1000 years after repository closure on total dose for the reasonable biosphere development is shown in *Figure 7-4*. The maximum total dose for a degraded Silo increases with a factor of three in comparison to intact barriers ( $9 \cdot 10^{-6}$  Sv/yr vs  $3 \cdot 10^{-6}$  Sv/yr). From 8000 AD and onwards this ratio increases to about an order of magnitude. It is the same nuclides that dominate the dose for both cases.

The main difference between an intact and a degraded barrier in BMA is that the release rate of inorganic <sup>14</sup>C increases up to a factor 20 with the higher water flow rate, and that <sup>239</sup>Pu and <sup>240</sup>Pu become two of the dose dominating nuclides from 8000 AD and onwards. For an intact BMA organic <sup>14</sup>C dominates the dose until 8000 AD (see Figure 5-14). However, when BMA has degraded after 1000 years organic and inorganic <sup>14</sup>C give about equal contributions to the maximum total dose at 5000 AD (see Figure 5-20). From that point, the dose from organic <sup>14</sup>C decreases why inorganic <sup>14</sup>C becomes the dose dominating nuclide between 5500 AD and 8000 AD. The maximum total dose for a degraded BMA is  $1 \cdot 10^{-6}$  Sv/yr which is a factor two higher than for an intact barrier in BMA.

Based on results from the hydrogeology modelling (Holmén and Stigsson, 2001a) it is estimated that the total dose for 1BTF and 2BTF should not increase with more than a factor two in comparison to the main case.

The total release from vaults and Silo will give a maximum total dose for the reasonable biosphere of  $1 \cdot 10^{-5}$  Sv/yr. For the alternative recipients the maximum total dose will be  $1 \cdot 10^{-8}$  Sv/yr (today's biosphere) and  $< 2 \cdot 10^{-5}$  Sv/yr (mire area), respectively. Release to a well from all vaults will give a maximum total dose of no more than  $5 \cdot 10^{-4}$  Sv/yr or from the Silo  $2 \cdot 10^{-4}$  Sv/yr.



*Figure 7-4.* Comparison of total dose for a reasonable biosphere from release of radionuclides in Silo and BMA when the repository is a) intact and b) degraded.

# 7.2 Initially degraded barriers

A calculation made for an initial fracture through the concrete encapsulation in room 12 of BMA shows that slightly higher near-field release rates can be obtained during the first 1000 years after repository closure compared to the release rates obtained when the fracture arises 1000 years after closure. The release rates for the two cases are almost the same after 1000 years. Thus, it is estimated that the maximum total dose will be the same regardless of if there is an initial fracture or if it arises after 1000 years. This will also be the case for a fracture through 1BTF or 2BTF. An initial fracture in the concrete bottom of the Silo is not foreseen to affect release rates from the Silo as long as the bentonite barriers are intact.

### 7.3 Chemicals

This scenario analyses the effect of a reduced possibility for nuclides to sorb on the barriers. The most important effect of chemicals is a significant increase in near-field release rate of <sup>59</sup>Ni, <sup>99</sup>Tc, <sup>239</sup>Pu and <sup>240</sup>Pu. <sup>59</sup>Ni and <sup>99</sup>Tc give a significant contribution to the dose for release to the reasonable biosphere (especially when the recipient is agricultural land), mire area and well. The maximum dose for the reasonable biosphere

development is obtained at 5000 AD, dominated by organic <sup>14</sup>C from the Silo. At 8000 AD the dose is dominated by <sup>99</sup>Tc from BMA at almost the same level as the maximum dose. Release to a well is dominated by the release from BMA. The maximum total dose is 4.10<sup>-4</sup> Sv/yr dominated by <sup>99</sup>Tc, <sup>239</sup>Pu and <sup>240</sup>Pu. Dose obtained in mire area by release from a repository uninfluenced by chemicals is entirely dominated by <sup>79</sup>Se. The same nuclide dominate the dose in mire area also for the scenario "Chemicals" but several other nuclides (e.g. <sup>59</sup>Ni, <sup>99</sup>Tc, <sup>239</sup>Pu and <sup>240</sup>Pu) add to the total dose. Today's biosphere is to a large extent unaffected by chemicals. Since sorption is not accounted for in BLA in any of the studied cases, no analysis of the effect of chemicals in BLA has been made. The same result is therefore used for BLA when estimating the total dose for SFR 1 with and without the effect of chemicals.

There is an increase in total dose for the reasonable biosphere from 8000 AD and onwards and for the wells, which mainly is caused by the higher release rate of <sup>99</sup>Tc. The dominating pathway for exposure from this nuclide is in both recipients the consumption of vegetables. Unfortunately, a too high translocation factor was used in the biosphere modelling of <sup>99</sup>Tc by mistake. This is only of importance when the well is used as recipient and the calculated doses are estimated to be about 20 % too high.



*Figure 7-5.* Comparison of total doses for different biospheres from release of radionuclides from an intact SFR 1 a) with (solid line) and b) without (dashed line) the effect of chemicals.

The combined effect of chemicals and degraded barriers after 1000 years in Silo and BMA gives a maximum dose for the reasonable biosphere of  $9 \cdot 10^{-6}$  Sv/yr and  $4 \cdot 10^{-6}$  Sv/yr, respectively. The corresponding dose for release to a mire area is  $7 \cdot 10^{-6}$  Sv/yr (Silo) and  $1 \cdot 10^{-5}$  Sv/yr (BMA), and for release to a well  $3 \cdot 10^{-4}$  Sv/yr (Silo) and  $9 \cdot 10^{-4}$  Sv/yr (BMA).

# 7.4 Far field

Transport through the far field (geosphere) delays the release of radionuclides to the biosphere. How effective the far field rock is as a barrier varies from nuclide to nuclide, but is also dependent on the effectiveness of the barriers in the near field. The influence of the far field on the release rate of the majority of the dose dominating nuclides is in general small. The far field has the largest effect on the release of short-lived nuclides like <sup>3</sup>H, <sup>63</sup>Ni, <sup>90</sup>Sr, <sup>108m</sup>Ag and <sup>137</sup>Cs. However, these nuclides have a limited influence on the maximum total dose for the different repository parts, with the exception of BLA.

The far field has a larger effect on the release of radionuclides from BLA to the biosphere than for the other repository parts. This is due to the limited technical barrier system in BLA. The far field delays the release, which for the cases with unchanged biosphere with time implies a lower dose. However, for the reasonable biosphere development the delay implies that the dose obtained for some nuclides is higher when the far field is included.

# 7.5 Permafrost

The analysis of the permafrost scenario gives a total dose that is less than  $10^{-9}$  Sv/yr assuming a recipient corresponding to today's biosphere. This dose is a factor ten lower than the maximum total dose for release to today's biosphere within the base scenario.

# 7.6 Human intrusion

The human intrusion scenario is represented by a case in which a well is sunk directly into one of the vaults or the Silo. It is assumed that someone drinks the water that is pumped out from the gravel backfill in each repository part. No dilution effects have been accounted for. The resulting doses are  $1 \cdot 10^{-2}$  Sv/yr (Silo),  $5 \cdot 10^{-3}$  Sv/yr (BMA),  $9 \cdot 10^{-3}$  Sv/yr (1BTF),  $1 \cdot 10^{-3}$  Sv/yr (2BTF) and  $3 \cdot 10^{-2}$  Sv/yr (BLA).

# 7.7 Summary of doses

The maximum doses obtained for the different cases studied are compiled in Table 7-1.

	Maxim							
Case	Silo	BMA	1BTF	2BTF	BLA	Total	Biosphere	
Base scenario								
Intact barriers	3 <sup>-</sup> 10 <sup>-6</sup>	5 <sup>.</sup> 10 <sup>-7</sup>	4 <sup>.</sup> 10 <sup>-7</sup>	9 <sup>.</sup> 10 <sup>-8</sup>	2 <sup>.</sup> 10 <sup>-7</sup>	4 <sup>.</sup> 10 <sup>-6</sup>	Reasonable biosphere development Mire area > 4000 AD Well >4000 AD	
	2 <sup>.</sup> 10 <sup>-7</sup>	3 <sup>.</sup> 10 <sup>-6</sup>	5 <sup>.</sup> 10 <sup>-7</sup>	7 <sup>.</sup> 10 <sup>-7</sup>	3 <sup>.</sup> 10 <sup>-6</sup>	6 <sup>.</sup> 10 <sup>-6</sup>		
	5 <sup>.</sup> 10 <sup>-5</sup>	6 <sup>.</sup> 10 <sup>-5</sup>	7 <sup>.</sup> 10 <sup>-5</sup>	1 <sup>.</sup> 10 <sup>-5</sup>	2 <sup>.</sup> 10 <sup>-4</sup>	4 <sup>.</sup> 10 <sup>-4 a)</sup>		
	7 <sup>.</sup> 10 <sup>-11</sup>	2 <sup>.</sup> 10 <sup>-10</sup>	3 <sup>.</sup> 10 <sup>-10</sup>	3 <sup>.</sup> 10 <sup>-11</sup>	1 <sup>.</sup> 10 <sup>-8</sup>	1 <sup>.</sup> 10 <sup>-8</sup>	Today's biosphere	
Degraded barriers	9 <sup>.</sup> 10 <sup>-6</sup>	1 <sup>.</sup> 10 <sup>-6</sup>	<u>≤</u> 8 <sup>·</sup> 10 <sup>-7 b)</sup>	≤ 2 <sup>.</sup> 10 <sup>-7 b)</sup>	2 <sup>.</sup> 10 <sup>-7 c)</sup>	1 <sup>.</sup> 10 <sup>-5</sup>	Reasonable biosphere development	
	5 <sup>.</sup> 10 <sup>-6</sup>	3 <sup>.</sup> 10 <sup>-6</sup>	<u>&lt;</u> 1 <sup>·</sup> 10 <sup>-6 b)</sup>	<u>&lt;</u> 1 <sup>.</sup> 10 <sup>-6 b)</sup>	3 <sup>.</sup> 10 <sup>-6 c)</sup>	<u>&lt;</u> 2 <sup>·</sup> 10 <sup>-5 b)</sup>	Mire area > 4000 AD	
	2 <sup>.</sup> 10 <sup>-4</sup>	7 <sup>.</sup> 10 <sup>-5</sup>	<u>&lt;</u> 1 <sup>.</sup> 10 <sup>-4 b)</sup>	<u>&lt;</u> 2 <sup>.</sup> 10 <sup>-5 b)</sup>	2 <sup>.</sup> 10 <sup>-4 c)</sup>	<u>&lt;</u> 5 <sup>·</sup> 10 <sup>−4 a,b)</sup>	Well >4000 AD	
	3 <sup>.</sup> 10 <sup>-10</sup>	2 <sup>.</sup> 10 <sup>-10</sup>	<u>≤</u> 10 <sup>-9 b)</sup>	<u>&lt;</u> 6 <sup>.</sup> 10 <sup>-11 b)</sup>	1 <sup>.</sup> 10 <sup>-8 c)</sup>	1 <sup>.</sup> 10 <sup>-8</sup>	Today's biosphere	
Other scenarios								
Initially degraded barriers	9 <sup>.</sup> 10 <sup>-6</sup>	1 <sup>.</sup> 10 <sup>-6</sup>	<u>≤</u> 8 <sup>·</sup> 10 <sup>-7 b)</sup>	<u>&lt;</u> 2 <sup>·</sup> 10 <sup>-7 b)</sup>	2 <sup>.</sup> 10 <sup>-7 c)</sup>	1 <sup>.</sup> 10 <sup>-5</sup>	Reasonable biosphere development	
	5 <sup>.</sup> 10 <sup>-6</sup>	3 <sup>.</sup> 10 <sup>-6</sup>	<u>&lt;</u> 1 <sup>.</sup> 10 <sup>-6 b)</sup>	<u>&lt;</u> 1 <sup>.</sup> 10 <sup>-6 b)</sup>	3 <sup>.</sup> 10 <sup>-6 c)</sup>	<u>&lt;</u> 2 <sup>.</sup> 10 <sup>-5 b)</sup>	Mire area > 4000 AD	
	2 <sup>.</sup> 10 <sup>-4</sup>	7 <sup>.</sup> 10 <sup>-5</sup>	<u>≤</u> 1 <sup>·</sup> 10 <sup>-4 b)</sup>	<u>&lt;</u> 2 <sup>·</sup> 10 <sup>-5 b)</sup>	2 <sup>.</sup> 10 <sup>-4 c)</sup>	<u>&lt;</u> 5 <sup>·</sup> 10 <sup>−4 a,b)</sup>	Well >4000 AD	
	3 <sup>.</sup> 10 <sup>-10</sup>	2 <sup>.</sup> 10 <sup>-10</sup>	<u>&lt;</u> 10 <sup>-9 b)</sup>	<u>&lt;</u> 6 <sup>.</sup> 10 <sup>-11 b)</sup>	1 <sup>.</sup> 10 <sup>-8 c)</sup>	1 <sup>.</sup> 10 <sup>-8</sup>	Today's biosphere	
Chemicals – intact barriers	3 <sup>.</sup> 10 <sup>-6</sup>	2 <sup>.</sup> 10 <sup>-6</sup>	4 <sup>.</sup> 10 <sup>-7</sup>	4 <sup>.</sup> 10 <sup>-7</sup>	2 <sup>.</sup> 10 <sup>-7 c)</sup>	4 <sup>.</sup> 10 <sup>-6</sup>	Reasonable biosphere development	
	4 <sup>.</sup> 10 <sup>-7</sup>	6 <sup>.</sup> 10 <sup>-6</sup>	1 <sup>.</sup> 10 <sup>-6</sup>	1 <sup>.</sup> 10 <sup>-6</sup>	3 <sup>.</sup> 10 <sup>-6 c)</sup>	1 <sup>.</sup> 10 <sup>-5</sup>	Mire area > 4000 AD	
	5 <sup>.</sup> 10 <sup>-5</sup>	4 <sup>.</sup> 10 <sup>-4</sup>	1 <sup>.</sup> 10 <sup>-4</sup>	8 <sup>.</sup> 10 <sup>-5</sup>	2 <sup>.</sup> 10 <sup>-4 c)</sup>	7 <sup>.</sup> 10 <sup>-4 a)</sup>	Well >4000 AD	
	7 <sup>.</sup> 10 <sup>-11</sup>	2 <sup>.</sup> 10 <sup>-10</sup>	3 <sup>.</sup> 10 <sup>-10</sup>	3 <sup>.</sup> 10 <sup>-11</sup>	1 <sup>.</sup> 10 <sup>-8 c)</sup>	1 <sup>.</sup> 10 <sup>-8</sup>	Today's biosphere	
Chemicals – degraded barriers	9 <sup>.</sup> 10 <sup>-6</sup>	4 <sup>.</sup> 10 <sup>-6</sup>	<u>&lt;</u> 8 <sup>·</sup> 10 <sup>−7 b)</sup>	<u>≤</u> 8 <sup>.</sup> 10 <sup>-7 b)</sup>	2 <sup>.</sup> 10 <sup>-7 c)</sup>	1 <sup>.</sup> 10 <sup>-5</sup>	Reasonable biosphere development	
	7 <sup>.</sup> 10 <sup>-6</sup>	1 <sup>.</sup> 10 <sup>-5</sup>	<u>&lt;</u> 2 <sup>·</sup> 10 <sup>−6 b)</sup>	<u>&lt;</u> 2 <sup>·</sup> 10 <sup>-6 b)</sup>	3 <sup>.</sup> 10 <sup>-6 c)</sup>	<u>&lt;</u> 2 <sup>·</sup> 10 <sup>-5 b)</sup>	Mire area > 4000AD	
	3 <sup>.</sup> 10 <sup>-4</sup>	9 <sup>.</sup> 10 <sup>-4</sup>	<u>&lt;</u> 2 <sup>·</sup> 10 <sup>-4 b)</sup>	<u>&lt;</u> 2 <sup>·</sup> 10 <sup>-4 b)</sup>	2 <sup>-10<sup>-4 c)</sup></sup>	<u>≤</u> 1 <sup>·</sup> 10 <sup>-3 a,b)</sup>	Well >4000 AD	
Permafrost	4·10 <sup>-10</sup>	3·10 <sup>-10</sup>	4·10 <sup>-10</sup>	6·10 <sup>-11</sup>	8·10 <sup>-11</sup>	10 <sup>-9</sup>	Today's biosphere	
Intrusion – well in repository	1.10 <sup>-2</sup>	5·10 <sup>-3</sup>	9·10 <sup>-3</sup>	1.10 <sup>-3</sup>	3·10 <sup>-2</sup>		Drinking well water	

Table 7-1.	Maximum	dose f	or the	investigated	cases.
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a) Total maximum dose for vaults

b) Estimated doses for 1BTF och 2BTF

c) Base scenario valid for this case also, since BLA has no barriers

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Appendix A: Check of screening of nuclides
The inventory for SFR comprises 59 radionuclides (Riggare and Johansson, 2001) but only 42 of these are considered to be of importance for the safety assessment (SKB, 2001). However, in order to reduce the time needed for calculation of radionuclide release and dose, the number of nuclides studied was reduced to 30. The inventory for <sup>241</sup>Pu was included in <sup>241</sup>Am, which simplifies the calculations and give a reasonable description of the release of both nuclides. This implies that calculations were performed for 29 nuclides.

Based on experience of the influence of sorption on the radionuclide release from other studies of for example SFR and SFL 3-5 totally 12 nuclides were excluded since they were expected to give a negligible contribution to the calculated total dose. The 12 nuclides that were excluded from the calculations are <sup>232</sup>U, <sup>234</sup>U, <sup>235</sup>U, <sup>236</sup>U, <sup>238</sup>U, <sup>237</sup>Np, <sup>242m</sup>Am, <sup>243</sup>Am, <sup>243</sup>Cm, <sup>244</sup>Cm, <sup>245</sup>Cm and <sup>246</sup>Cm. In this Appendix this expectation is checked.

The steps in the check are:

- Looking at the initial inventory and influence of chain decay. If, during the studied time period, significant contribution from mother(s) is achieved the whole contribution were included as initial inventory.
- Based on the sorption data reference nuclides were chosen. The calculated dose from the reference nuclide in relation to the total dose for the different calculation cases and biospheres are used to estimate whether the actual nuclide contribute to the total dose or not.
- A relative dose was calculated, i.e. the dose calculated from ingestion of the initial inventory (including the contribution from mother) for the nuclide in comparison to the reference nuclide.
- Possible difference in uptake mechanisms in the biosphere of the actual nuclide and the reference nuclide was controlled by comparison between dose conversion factors from SR 97.

The influence of chain decay is negligible for most of the 12 nuclides, since they are either mothers or daughters to long-lived mothers. This concerns <sup>232</sup>U, <sup>235</sup>U, <sup>238</sup>U, <sup>242m</sup>Am, <sup>243</sup>Am, <sup>243</sup>Cm, <sup>244</sup>Cm, <sup>245</sup>Cm and <sup>246</sup>Cm. For three nuclides the contribution from mothers must be included. For <sup>234</sup>U contribution from <sup>238</sup>Pu results in a 120 % higher initial inventory if all the decay of the mothers is included in the initial inventory. For <sup>236</sup>U the contribution from the mother is totally 60 % during all time. The contribution to <sup>237</sup>Np is 50 % for the tunnels and 400 % for the Silo.

The choice of reference nuclide is based on sorption data, see Table A-1. As can be seen in the table uranium and neptunium have sorption data equal to plutonium in concrete/cement as well as in sand/gravel. This implies that <sup>240</sup>Pu is chosen as reference for BMA, 1BTF and 2BTF. Although no sorption is accounted for in BLA, <sup>240</sup>Pu is chosen as reference also for this tunnel. The sorption in bentonite is, however, lower for uranium and neptunium than plutonium why another reference nuclide must be chosen for the Silo. In order not to overestimate the sorption capacity <sup>126</sup>Sn is chosen. Sorption data for curium is equal to americium and hence <sup>241</sup>Am is suitable as reference nuclide both for curium and americium nuclides, but since the half-life of <sup>241</sup>Am is shorter than for some of these nuclides it is only used as reference for <sup>242m</sup> Am, <sup>243</sup>Cm and <sup>244</sup>Cm. For the three remaining nuclides <sup>243</sup>Am, <sup>243</sup>Cm and <sup>246</sup>Cm the reference nuclide was chosen to be <sup>93</sup>Zr, even though the sorption is less in all barriers. The performed

radionuclide release calculations show a negligible contribution from the reference nuclides <sup>126</sup>Sn, <sup>241</sup>Am and <sup>93</sup>Zr on the total dose rate for all repository parts and biospheres. The reference nuclide <sup>240</sup>Pu contributes to the total dose rate for BLA when the recipient is a well, but not for any of the other repository parts or biospheres studied.

The calculated relative dose, i.e. the dose calculated from dose conversion factors for ingestion (Table A-1 (EU, 1996)) and the initial inventory (including the contribution from mother) for the nuclide in comparison to the reference nuclide, are shown in Table A-2. As can be seen in the table <sup>234</sup>U, <sup>236</sup>U, <sup>238</sup>U and <sup>237</sup>Np in the Silo exceed the dose from the reference nuclide. However, the choice of <sup>126</sup>Sn as reference nuclide implies that the sorption in the barriers is underestimated. The dose from <sup>126</sup>Sn from the Silo is small in comparison to the total dose and hence also the nuclides compared to <sup>126</sup>Sn have negligible influence on the total dose. For BLA the nuclides compared to <sup>240</sup>Pu show several orders of magnitude lower relative dose and hence even if <sup>240</sup>Pu contribute to the total dose in a well these nuclides will not. The relative dose for <sup>243</sup>Am and <sup>245</sup>Cm indicate a higher dose than the dose from the reference nuclide <sup>93</sup>Zr. Again, the reference nuclide sorbs less in all barriers and in the performed radionuclide release calculations for <sup>93</sup>Zr the dose only exceeds 1/10 000 in the cases with influence of chemicals and for BMA with degraded barriers. The influence of chemicals is however less for americium and curium than for zirconium. This implies that <sup>243</sup>Am may contribute to some extent to the total dose and therefore a radionuclide release calculation for BMA for the case with chemicals and degraded barriers was performed. No biosphere calculations were performed, but comparison to <sup>239</sup>Pu and <sup>93</sup>Zr shows that it may be possible that the dose from <sup>243</sup>Am is only one order of magnitude less than the total dose and hence would contribute marginally to the total dose.

The dose conversion factors used in SR97 (Nordlinder et al, 1999) for different ecosystems, coast, open coast, agricultural soil, peat and small peat was used to check that the uptake mechanisms in the biosphere for the reference nuclide and actual nuclide are similar. This comparison showed that the change in dose conversion factor compared to the reference nuclide is within a factor of thirty for all nuclides and ecosystems. The consequence of an increase of a factor 30 if one of the 12 nuclides is released to an ecosystem that implies higher relative dose than for the reference nuclide will not lead to a dose that contribute to the total dose, except maybe for <sup>243</sup>Am. This implies that the exclusion of these 12 nuclides does not influence the total dose, except for <sup>243</sup>Am that maybe would contribute marginally to the dose in some case for BMA.

Nuclide	Half life	Dose	Distribution coefficients (m <sup>3</sup> /kg)			
_	(yr)	conversion factor, ingestion (Sv/Bq)	Concrete/ Cement	Sand/ gravel	Bentonite	bentonite/ sand 10/90
Zr-93	1.53.10 <sup>6</sup>	1.10·10 <sup>-9</sup>	0.5	0.5	0.05	0.5
Sn-126	1.00·10 <sup>5</sup>	4.70·10 <sup>-9</sup>	0.5	0	0.01	0.001
Pu-240	6.56·10 <sup>3</sup>	2.50·10 <sup>-7</sup>	5	1	1	1
Am-241	$4.32 \cdot 10^2$	2.00·10 <sup>-7</sup>	1	1	1	1
U-232	6.89·10 <sup>1</sup>	3.30·10 <sup>-7</sup>	5	1	0.01	0.9
U-234	2.46·10 <sup>5</sup>	4.90·10 <sup>-8</sup>	5	1	0.01	0.9
U-235	7.04·10 <sup>8</sup>	4.70·10 <sup>-8</sup>	5	1	0.01	0.9
U-236	2.34·10 <sup>7</sup>	4.70·10 <sup>-8</sup>	5	1	0.01	0.9
U-238	4.47·10 <sup>9</sup>	4.50·10 <sup>-8</sup>	5	1	0.01	0.9
Np-237	2.14·10 <sup>6</sup>	1.10·10 <sup>-7</sup>	5	1	0.1	0.9
Am-242m	1.41·10 <sup>2</sup>	1.90·10 <sup>-7</sup>	1	1	1	1
Am-243	7.37.10 <sup>3</sup>	2.00·10 <sup>-7</sup>	1	1	1	1
Cm-243	2.91·10 <sup>1</sup>	1.50·10 <sup>-7</sup>	1	1	1	1
Cm-244	1.81·10 <sup>1</sup>	1.20·10 <sup>-7</sup>	1	1	1	1
Cm-245	8.50·10 <sup>3</sup>	2.10·10 <sup>-7</sup>	1	1	1	1
Cm-246	4.73·10 <sup>3</sup>	2.10·10 <sup>-7</sup>	1	1	1	1

Table A-1. Dose conversion factors (Sv/Bq) and distribution coefficients (m<sup>3</sup>/kg)

 Table A-2. Calculated relative doses for all repository parts.

Nuclide	Reference nuclide	Rela Silo	ative dose co BMA	ompared to " BTF1	reference" n BTF2	uclide BLA
U-232	Tunnels: Pu-240 Silo: Sn-126	0.6	0.00005	0.00004	0.00005	0.00005
U-234	Tunnels: Pu-240 Silo: Sn-126	8	0.0006	0.0006	0.0006	0.0006
U-235	Tunnels: Pu-240 Silo: Sn-126	0.07	0.00001	0.00001	0.00001	0.00001
U-236	Tunnels Pu-240 Silo: Sn-126	2	0.0001	0.0001	0.0001	0.0001
U-238	Tunnels: Pu-240 Silo: Sn-126	1.3	0.0001	0.0001	0.0001	0.0001
Np-237	Tunnels: Pu-240 Silo: Sn-126	16	0.0004	0.0004	0.0004	0.0004
Am-242m	Am-241	0.001	0.009	0.009	0.009	0.009
Am-243	Zr-93	216	113	111	49	354
Cm-243	Am-241	0.001	0.009	0.008	0.009	0.009
Cm-244	Am-241	0.1	0.8	0.7	0.7	0.8
Cm-245	Zr-93	2	1.2	1.2	0.5	4
Cm-246	Zr-93	0.6	0.3	0.3	0.1	1.0

## References

EU (1996) Rådets direktiv 96/29/Euratom av den 13 maj, 1996, Published in the European community's official journal, L 159, June 29, 1996 (in Swedish).

Nordlinder S., Bergström U. and Mathiasson L. (1999) Ecosystem specific dose conversion factors for Aberg, Beberg and Ceberg, SKB Technical report TR-99-15, Swedish Nuclear and Fuel Waste Management Co, Stockholm, Sweden.

Riggare P. and Johansson C. (2001) Project SAFE Low and Intermediate Level Waste in SFR-1, SKB report R-01-03, Swedish Nuclear and Fuel Waste Management Co, Stockholm, Sweden.

SKB (2001) Project SAFE – Compilation of data for radionuclide transport analysis, SKB report R-01-14, Swedish Nuclear and Fuel Waste Management Co, Stockholm, Sweden.

# Appendix B: Discretization in near field models

Schematic figures of the discretizations used for the near field models used for the different repository parts.

#### Modell of the SiLO



11

- 58 Concrete lid
- 51 Concrete bottom
- 52 Sand/Bentonite bottom
- 53-54 Fictitious compartments



8 9

7

#### Modell of BMA

128 127 128 82 81	111 129 129 129 129 125 54 54 54 54 54 54 54 54 54 5	134 133 133 133 133 133 133 133	1107 1106 1106 72 71 70 120 120 120 120 120 120 120 12	140 129 129 129 129 120 72 74 72 74 72 73 60 60 60 60	140 142 141 70 77 76 122 10 10 10 10 10 10 10 10 10 10	123 134 125 189 166 191
	144         500         64           182         500         64           182         100         65           140         154         64	145 (145 (145 100 100 101 102 104 105 161	14g	142 00 100 94 100 95 110 100	1481 1481 111 112 112 112 112 113	Gravel Concrete Concrete Waste Fictitious compartment
1-2, 11-12, 21-22, 31-32, 41-42 3-4, 13-14, 22-24, 33-34, 43-44 25-26, 35-36, 45-46 3-8, 17-18, 37-38 9-10, 19-20 5-6, 15-16, 27-30, 39-40, 47-50	Waste stabilised with cement in concrete moulds Waste stabilised with cement in steel containers Waste stabilised with cement in steel drums Waste stabilised with bitumen in steel containers Waste stabilised with bitumen in steel drums Fictibious source terms	61-53 Wall at short side of the structure ( 64-78 Ceeing of the structure 79-83 Bottom of the structure 84-98 Wall on left side of the structure 90-112 Wall on right side of the structure 114 Inner well between room 4 and 5	(passage) 116 Inner wall between 117 Inner wall between 118 Inner wall between 119 Inner wall between 120 Inner wall between 121 Fictbous campan	room 11 and 12 room 12 and 13 room 1 and 2, 2 and 3, 3 and 4 (sink) room 5 and 6, 6 and 7, 7 and 8, 6 an room 10 and 11 (sink) meet (No inver wall, only one room)	123-125 Wall at sho 125-126 Gravel at th 129-143 Gravel at th 144-148 Gravel at th 146-153 Lateral gra 154-156 Lateral gra	t side of the structure (loading zone) e end of the tunnel (passage) e top of the tunnel bottom of the tunnel wel on the left side of the tunnel wel on the left side of the tunnel

- 5-6, 15-16, 27-30, 39-40, 47-50 Fictitious source terms
- 51-55 Walls of concrete moulds
- 56-80 Water inside encapsulation
- 115 Inner wall between room 9 and 10
- 121 Fictitious compartment (No inner wall, only one room) 154-158 Lateval gravel on the right side of the turnel
- 122 Inner wall between room 13 and 14/15 and between room 14 and 15 (sin 159-101 Gravel at the end of the turnel (loading zone)

B:2



#### Modell of 1BTF



#### Modell of 2BTF

B:4

## Modell of BLA



1-5 Source term

6-7 Gravel at the end of the tunnel

# Appendix C: Examples of input files for near-field calculations

Examples of input files for the organic <sup>14</sup>C in the main case in the base scenario for all repository parts

SILO CALCULATIONS Problem Codification: BS2 Nuclide(s): C-14 org # # Problem Description: # PROPER VERSION INPUT DATA FORMAT NGROUP NSINKS IWFLOW NSOURC NBSIZE ISPSOL # TTYPE 3 18 0 0 1 1 1 # Flow stream conditions # NINFLW NUTFLW % if TWFLOW = 1 10 1 Inflow conditions (one line per inflow) # OINFW NBWIN NBNWIN % if IWFLOW = 1 # 0.00005 5 1 0.00015 11 1 0.00005 17 1 0.044 43 1 0.044 44 1 0 044 45 1 0 044 46 1 0.044 47 1 0 038 49 1 0.23 52 1 Outflow conditions (one line per inflow) # OUTFW NBWOUT NBNWOUT % if IWFLOW = 1 0.48825 43 1 # Number of blocks connections and materials # NBLOCK NBCOUP NMAT 54 66 10 # Data on materials (one line per material) # Material 1 = Waste (Concrete moulds with cement) # Material 2 = Waste (Steel with cement) # Material 3 = Waste (Bitumen) # Material 4 = Porous concrete # Material 5 = Construction concrete (moulds silo internal) # Material 6 = Bentonite (outside silo walls) # Material 7 = Gravel # Material 8 = Sand/bentonite (bottom and top) # Material 9 = Sand (top) # Material 10 =Water # MAT DENSM PORM 2250.0 0.295 1 2250.0 2 0.173 3 1030.0 0.119 2428.6 4 0.300 2529.4 5 0.150 6 2692.3 0.610 7 2700.0 0.300 2666.7 8 0.250 2700 0 9 0.300 10 1000.0 1.000 Data on time series # TINIT AGEO RAT TEND LREAD NTERM # 0.00 0.50 1.23 1000.00 90 0 # # Data on numerical parameters # EPS EWT NLOOP MSPAR

1.00E-06 1.00E-18 3 1 # Data on group (one set of data per group common for all sources) # Group No: 1 NNCH ISPEC ISPSOL # 1 0 1 # Nuclide: C-14 org # TAU CSOLUB ATNV 5.7300E+03 1.17E+01 7.6054E-01 % if TSPEC = 1 # CIRE # # МΔТ DEFM SKDM % one line per material 0.00315 0 000 1 0.00315 0.000 2 0.06307 0 000 3 4 0.00315 0.000 0.00032 5 0.000 6 0.00315 0.000 0.01892 0.000 7 0.00315 0.000 8 9 0.01892 0.000 10 0.06307 0.000 Data on sources # NSBCHA ISEQ MAT IFWATB VOLSOU FINVEN JINVEN # ISSEO % if ISEQ = 1 # IBLOCK XXX % if JINVEN = 1. One line for each group. # % if ISEO = 0 TBSOUR # # IWINT % if IFWATB = 1 WBFLOW % if IFWATB = 1 and IWINT = 0 Waste Source 1 1 0 1 1 702.2400 1.0000000E+00 1 4.6663405E-02 1 0 1.868278E-01 # Source 2 1 0 1 1 175.5600 1.000000E+00 1 1.1665851E-02 2 0 1.868278E-01 # Source 3 1 0 2 1 1053.5504 1.0000000E+00 1 5.1720429E-02 3 0 1.868278E-01 # Source 4 1 0 2 1 263.3876 1.0000000E+00 1

```
1.2930107E-02
 4
 0
 1.868278E-01
# Source 5
 1 0 3 1 0.0001 1.0000000E+00 1
 7.7020207E-02
 5
 0
 5.000000E-05
# Source 6
 1 0 3 1 617.0864 1.0000000E+00 1
#
 0.000000E+00
 6
 0
5.000000E-05
# Source 7
1 0 1 1 2106.7200 1.0000000E+00 1
#
1.3999022E-01
 7
 0
 1.868278E-01
# Source 8
 1 0 1 1 526.6800 1.000000E+00 1
#
 3.4997554E-02
 8
 0
 1.868278E-01
# Source 9
 1 0 2 1 3160.6512 1.0000000E+00 1
#
1.5516129E-01
 9
 0
1.868278E-01
# Source 10
 1 0 2 1 790.1628 1.0000000E+00 1
 3.8790322E-02
 10
 0
1.868278E-01
# Source 11
 1 0 3 1 0.0002 1.000000E+00 1
#
 2.3106062E-01
 11
 0
 1.500000E-04
# Source 12
1 0 3 1 1851.2592 1.0000000E+00 1
#
 0.000000E+00
 12
 0
 1.500000E-04
# Source 13
 1 0 1 1 702.2400 1.0000000E+00 1
#
 4.6663405E-02
```

```
13
 0
 1.868278E-01
# Source 14
 1 0 1 1 175.5600 1.000000E+00 1
 1.1665851E-02
 14
 0
 1.868278E-01
# Source 15
 1 0 2 1 1053.5504 1.0000000E+00 1
#
 5.1720429E-02
15
0
 1.868278E-01
# Source 16
 1 0 2 1 263.3876 1.0000000E+00 1
#
 1.2930107E-02
 16
 0
 1.868278E-01
# Source 17
 1 0 3 1 0.0001 1.0000000E+00 1
#
 7.7020207E-02
 17
 0
 5.000000E-05
# Source 18
 1 0 3 1 617.0864 1.0000000E+00 1
#
 0.000000E+00
 18
 0
 5.000000E-05
#
# Description of blocks not considered before
# Block i
# NZ NY NX NBLSP MAT IOFLOW
# ZADIM YADIM XADIM VOLBLK % one line per compartment if NBLSP = 1
# IWINT
                         % if IOFLOW = 1
# NODWIN NODWUT BFLOW % if IQFLOW = 1 and IWINT = 0
# Porous concrete in 'inner cylinder' (bitumenised waste)
# Block 19
 1 1 1 0 4 1
 3.8819E-04 9.6177E-05
                           3.0461E-05
                                         1.9049E+02
 1
# Block 20
 1 1 1 0 4 1
 1.2940E-04 3.2059E-05
                           1.0154E-05
                                         5.7146E+02
 1
# Block 21
 1 1 1 0 4 1
 3.8819E-04 9.6177E-05
                           3.0461E-05
                                         1.9049E+02
 1
# Porous concrete in 'outer cylinder' (cement conditioned waste)
# Block 22
1 1 1 0 4 1
 5.4227E-04 3.0888E-05
                           2.8910E-05
                                         8.2432E+02
```

1 # Block 23 1 1 1 0 4 1 5.5153E-04 1.8076E-04 9.6366E-06 2 4730E+03 1 # Block 24 1 1 1 0 4 1 5.4227E-04 3.0888E-05 2.8910E-05 8.2432E+02 1 # Walls of concrete moulds # Block 25 1 1 1 0 5 0 1.0000E+05 6.8051E-06 1.5563E-05 6.3904E+02 # Block 26 1 1 1 0 5 0 1.0000E+05 2.2684E-06 5.1877E-06 1.9171E+03 # Block 27 1 1 1 0 5 0 1.0000E+05 6.8051E-06 1.5563E-05 6.3904E+02 # Internal walls in 'inner cylinder' (bitumenised waste) # Block 28 1 1 1 0 5 0 1.0000E+05 1.0000E+05 6.7693E-05 1.4773E+02 # Block 29 1 1 1 0 5 0 1.0000E+05 1.0000E+05 2.2564E-05 4.4318E+02 # Block 30 1 1 1 0 5 0 1.0000E+05 1.0000E+05 6.7693E-05 1.4773E+02 # Internal walls in 'outer cylinder' (cement conditioned waste) # Block 31 1 1 1 0 5 0 1.0000E+05 1.0000E+05 1.5643E-05 6.3927E+02 # Block 32 1 1 1 0 5 0 1.0000E+05 1.0000E+05 5.2143E-06 1.9178E+03 # Block 33 1 1 1 0 5 0 1.0000E+05 1.0000E+05 1.5643E-05 6.3927E+02 # Walls between bitumenised waste and cement conditioned # waste ('inner cylinder' and 'outer cylinder') (34-36) # Block 34 1 1 1 0 5 0 1.0000E+05 5.4645E-04 5.4645E-04 7.3200E+01 # Block 35 1 1 1 0 5 0 1.0000E+05 1.8215E-04 1.8215E-04 2.1960E+02 # Block 36 1 1 1 0 5 0 1.0000E+05 5.4645E-04 5.4645E-04 7.3200E+01 # Concrete silo walls (37-39) # Block 37 1 5 1 0 5 0 1.0000E+05 9.2957E-04 1.0000E+05 6.8849E+02 # Block 38 1 5 1 0 5 0 1.0000E+05 3.0986E-04 1.0000E+05 2.0655E+03 # Block 39 1 5 1 0 5 0 1.0000E+05 9.2957E-04 1.0000E+05 6.8849E+02 # Bentonite silo walls (40-42) # Block 40 151060 1.0000E+05 1.2485E-03 1.0000E+05 1.2183E+03

# Block 41 151060 1.0000E+05 4.1617E-04 1.0000E+05 3 3303E+03 # Block 42 151060 1.0000E+05 1.2485E-03 1.0000E+05 1.2183E+03 # Gravel at the top (43-47) # Block 43 1 1 1 0 7 1 2.1342E-03 1.0000E+05 1.0000E+05 1.2158E+03 1 # Block 44 1 1 1 0 7 1 2.1342E-03 1.0000E+05 1.0000E+05 1.2158E+03 1 # Block 45 1 1 1 0 7 1 2.1342E-03 1.0000E+05 1.0000E+05 1.2158E+03 1 # Block 46 1 1 1 0 7 1 2.1342E-03 1.0000E+05 1.0000E+05 1.2158E+03 1 # Block 47 1 1 1 0 7 1 2.1342E-03 1.0000E+05 1.0000E+05 1.2158E+03 # Sand/Bentonite at top (48) # Block 48 5 1 1 0 8 1 1.9874E-03 1.0000E+05 1.0000E+05 1.1322E+03 0 5 0 268 1 # Sand at top (49) # Block 49 1 1 1 0 9 1 1.3249E-04 9.2424E-04 1.6836E-04 7.5477E+01 1 # Concrete silo top (50) # Block 50 5 1 1 0 5 1 1.6836E-03 1.0000E+05 1.0000E+05 5.9396E+02 0 5 1 0.23 # Concrete silo bottom (51) # Block 51 5 1 1 0 5 1 1.6836E-03 1.0000E+05 1.0000E+05 5.9396E+02 0 1 0.23 5 # Sand/Bentonite at bottom (52) # Block 52 5 1 1 0 8 1 2.5254E-03 1.0000E+05 1.0000E+05 1.0532E+03 0 5 1 0.23 # Fictitious blocks (53-54) # Block 53 1 1 1 0 10 1 1.0000E-05 1.0000E-05 1.0000E-05 1.0000E-02 1 # Block 54 1 1 1 0 10 1 1.0000E-05 1.0000E-05 1.0000E-05 1.0000E-02

```
1
  Codifying the connections
#
# NSSCO NSBCO
                  NBBCO
 9 18 39
#
  Connection between sources
  NABCO ICSEO
                  LSCFW IRADD
#
#
  QWATIN
                  % if LSCFW is not equal to 0
  ISERNU
                  % if TRADD = 1
#
#
#
  XRADD
                  % if IRADD = 1 and ISERNU = 1 one line per group
#
                  % if IRADD = 1 and ISERNU = 0 one line per group
#
 XRADD
# IBSA IBSB ISEOA ISEOB % if NABCO > 1 and ICSEO = 1
#
# IBSA IBSB
                 % if ICSEO = 0 NABCO*lines
#
  WITHIN WASTE
#
#
  1 Source-source 1 - 2
    1
                   0
                                 1
                                                1
    1.868278E-01
    1
    9.131667E-03
                   2
    1
# 2 Source-source 3 - 4
                                                1
    1
                   0
                                 1
    1.868278E-01
    1
    8.722704E-03
    3
                   Δ
# 3 Source-source 5 - 6
    1
                   0
                                 1
                                                 1
    5.000000E-05
    1
    1.000000E+12
    5
                   6
# 4 Source-source 7 - 8
    1
                   0
                                  1
                                                 1
    1.868278E-01
    1
    3.043889E-03
    7
                   8
# 5 Source-source 9 - 10
    1
                   0
                                 1
                                                1
    1.868278E-01
    1
    2.907568E-03
    9
                   10
# 6 Source-source 11 - 12
    1
                   0
                                  1
                                                1
    1.500000E-04
    1
    1.000000E+12
    11
                    12
# 7 Source-source 13 - 14
    1
                   0
                                 1
                                                1
    1.868278E-01
    1
```

```
9.131667E-03
    13
                    14
 8 Source-source 15 - 16
    1
                   0
                                  1
                                                 1
    1.868278E-01
    1
    8.722704E-03
    15
                    16
  9 Source-source 17 - 18
    1
                   0
                                  1
                                                 1
    5.00000E-05
    1
    1.000000E+12
    17
                    18
  Connection source - block
  IBSOU IBSB ISEOC NABCCO IRZB ICRB IPLUG IRADD
#
  ICOMPB ICSEQB % if NABCCO > 1 and ISEQC =1
#
                  % if ISEOC = 0. One value for each connection.
  TDCB
#
#
  LSCFW OWATIN
                 % if TEWATB(source term) = 1 and TOFLOW(block) = 1
#
  ISERNU
                  % if IRADD =1
                  % if IRADD = 1 and ISERNU = 1
  XRADD
#
                  % if IRADD = 1 and ISERNU = 0. One line per group.
#
  XRADD
  Between waste and conrete walls of concrete moulds
#
  1 Source-block 2 - 25
#
    2 25 0 1 1 1 0 1
  1
 1
 0.00114222934111067
# Between cement waste and porous concrete
# 2 Source-block 4 - 22
    4 22 0 1 2 1 0 1
  1
 1 0.186827771355237
 1
 0.00109109676399092
# Between bitumen waste - porous concrete
# 3 Source-block 6 - 19
    6 19 0 1 2 1 0 1
 1
 1 0.00005
 1
 6.55058132211069E-05
# Between waste and conrete walls of concrete moulds
  4 Source-block 8 - 26
    8 26 0 1 1 1 0 1
 1
 1
 0.00114222934111067
# Between cement waste and porous concrete
# 5 Source-block 10 - 23
    10 23 0 1 2 1 0 1
  1
 1 0.186827771355237
 1
```

```
0.00109109676399092
# Between bitumen waste - porous concrete
# 6 Source-block 12 - 20
   12 20 0 1 2 1 0 1
 1
 1 0.00015
 1
 6.55058132211069E-05
# Between waste and conrete walls of concrete moulds
# 7 Source-block 14 - 27
   14 27 0 1 1 1 0 1
 1
 1
 0.00114222934111067
# Between cement waste and porous concrete
# 8 Source-block 16 - 24
   16 24 0 1 2 1 0 1
 1
 1 0.186827771355237
 1
0.00109109676399092
# Between bitumen waste - porous concrete
# 9 Source-block 18 - 21
   18 21 0 1 2 1 0 1
 1 0.00005
 6.55058132211069E-05
# Between waste in concrete containers - porous concrete
# 10 Source-block 1 - 22
  1 22 0 1 1 0 0 1
 1
 -1 0.186827771355237
 1
 100000
# 11 Source-block 2 - 22
  2 22 0 1 1 0 0 1
 1
 1 0.186827771355237
 1
 100000
# 12 Source-block 7 - 23
  7 23 0 1 1 0 0 1
 1
 -1 0.186827771355237
 1
100000
# 13 Source-block 8 - 23
   8 23 0 1 1 0 0 1
 1
 1 0.186827771355237
 1
 100000
# 14 Source-block 13 - 24
   13 24 0 1 1 0 0 1
 1
 -1 0.186827771355237
 1
 100000
# 15 Source-block 14 - 24
   14 24 0 1 1 0 0 1
 1
 1 0.186827771355237
```

1 100000 # 16 Source-block 3 - 22 3 22 0 1 1 0 0 1 1 -1 0.186827771355237 1 100000 # 17 Source-block 9 - 23 9 23 0 1 1 0 0 1 1 -1 0.186827771355237 1 100000 # 18 Source-block 15 - 24 15 24 0 1 1 0 0 1 1 -1 0.186827771355237 1 100000 # Connections between blocks not considered as sources # IBA IBB ISEOC NABCCO IRZA IRZB ICRA ICRB IPLUG IRADD # TDCA TDCB % if NABCCO = 1 ICOMPA ICOMPB ICSEQA ICSEOB % if NABCCO > 1 and ISOC = 1 IDCA % if NABCCO > 1 and ISOC = 0 # % if NABCCO > 1 and ISOC = 0 # IDCB % if IOFLOW(A) = 1 and IOFLOW(B) = 1 # LSCFW OWATIN # ISERNU % if IRADD = 1 # XRADD % if IRADD = 1 and ISERNU = 0. One line per group # Within porous concrete in 'inner cylinder' (bitumenised waste) # 1 Block-block 19 - 20 19 20 0 1 2 2 0 0 0 1 1 1 -1 4.31722E-02 1 5.4240E-01 # 2 Block-block 20 - 21 20 21 0 1 2 2 0 0 0 1 1 1 -1 4.31722E-02 1 5.4240E-01 # Between porous concrete and internal concrete walls in 'inner cylinder' (bitumenised waste) # 3 Block-block 19 - 28 19 28 0 1 1 2 1 1 0 0 1 1 # 4 Block-block 20 - 29 20 29 0 1 1 2 1 1 0 0 1 1 # 5 Block-block 21 - 30 1 1 2 1 1 0 0 21 30 0 1 1 # Between porous concrete and concrete walls between 'inner cylinder' (bitumenised waste) and 'outer cylinder' (cement conditioned waste)

# Between sand in top of silo concrete 'lid' of silo # 36 Block-block 49 - 50 49 50 0 1 2 0 1 1 0 0 1 1 -1 2.30000E-01 # Between concrete in bottom of silo and sand/bentonite in bottom of silo # 37 Block-block 51 - 52 51 52 0 1 0 0 1 1 0 0 5 1 -1 2.30000E-01 # Between concrete in top of silo and fictitious block # 38 Block-block 50 - 53 50 53 0 1 0 0 0 1 0 0 5 1 -1 2.30000E-01 # Between concrete in bottom of silo and fictitious block # 39 Block-block 51 - 54 51 54 0 1 0 0 0 1 0 0 1 1 1 2.30000E-01 #Data connecting the sinks with the system: one line per sink # QEQ IBS ICS IRZ ICRS ZAFRA 
 0.00351302855477305
 40
 5
 1
 0

 0.0185983864664455
 41
 5
 1
 0

 0.020251576374574
 42
 5
 1
 0

# BMA CALCULATIONS # Problem Codification: BS1 # Nuclide(s): C-14org # Problem Description: PROPER VERSION INPUT DATA FORMAT # ITYPE NGROUP NSINKS IWFLOW NSOURC NBSIZE ISPSOL # 1 1 0 1 50 0 0 # Flow stream conditions # NINFLW NUTFLW % if IWFLOW = 1 21 26 # Inflow conditions (one line per inflow) # OINFW NBWIN NBNWIN % if IWFLOW = 1 2.005571E-05 7 1 3.775902E-05 9 1 5.236769E-05 17 1 3.226363E-05 19 1 5.000000E-05 37 1 2.855333E-02 126 1 2.855333E-02 127 1 2.855333E-02 128 1 8.611880E-01 144 1 1.099550E+00 145 1 4.603700E-01 146 1 2.961630E+00 147 1 2.238840E+00 148 1 8.086000E-02 154 1 1.654200E-01 155 1 4.641000E-02 156 1 8.027000E-02 157 1 4.559000E-02 158 1 3.602733E-01 159 1 3.602733E-01 160 1 3.602733E-01 161 1 # Outflow conditions (one line per inflow) # OUTFW NBWOUT NBNWOUT % if IWFLOW = 1 9.072333E-02 126 1 9.072333E-02 127 1 3.042833E-01 128 1 1.070633E-01 129 1 1.070633E-01 130 1 1.431943E+00 131 1 1.225267E-01 132 1 1.225267E-01 133 1 1.671747E+00 134 1 5.164667E-02 135 1 5.164667E-02 136 1 7.183667E-01 137 1 5.356667E-02 138 1 5.356667E-02 139 1 9.312967E-01 140 1 4.536667E-02 141 1 4.536667E-02 142 1 5.948067E-01 143 1 1.549200E-01 149 1 1.697100E-01 150 1 6.824000E-02 151 1 7.149000E-02 152 1 4.544000E-02 153 1

6.097567E-01 159 1 6.097567E-01 160 1 8.832567E-01 161 1 # Number of blocks connections and materials NBLOCK NBCOUP NMAT # 161 216 28 Data on materials (one line per material) # Material 1 = Waste (Concrete mould w. Cement in room 1-4) # Material 2 = Waste (Steel container w. Cement in room 1-4) # Material 3 = Waste (Steel drum w. Cement in room 1-4) (not used) # Material 4 = Waste (Steel container w. Bitumen in room 1-4) # Material 5 = Waste (Steel drum w. Bitumen in room 1-4) # Material 6 = Waste (Concrete mould w. Cement in room 5-9) # Material 7 = Waste (Steel container w. Cement in room 5-9) # Material 8 = Waste (Steel drum w. Cement in room 5-9) (not used) Material 9 = Waste (Steel container w. Bitumen in room 5-9) Material 10 = Waste (Steel drum w. Bitumen in room 5-9) Material 11 = Waste (Concrete mould w. Cement in room 10-11) # Material 12 = Waste (Steel container w. Cement in room 10-11) # Material 13 = Waste (Steel drum w. Cement in room 10-11) # Material 14 = Waste (Steel container w. Bitumen in room 10-11) (not used) # Material 15 = Waste (Steel drum w. Bitumen in room 10-11) (not used) # Material 16 = Waste (Concrete mould w. Cement in room 12) # Material 17 = Waste (Steel container w. Cement in room 12) # Material 18 = Waste (Steel drum w. Cement in room 12) # Material 19 = Waste (Steel container w. Bitumen in room 12) # Material 20 = Waste (Steel drum w. Bitumen in room 12) (not used) # Material 21 = Waste (Concrete mould w. Cement in room 13-15) # Material 22 = Waste (Steel container w. Cement in room 13-15) # Material 23 = Waste (Steel drum w. Cement in room 13-15) # Material 24 = Waste (Steel container w. Bitumen in room 13-15) (not used) # Material 25 = Waste (Steel drum w. Bitumen in room 13-15) (not used) # Material 26 = Construction concrete # Material 27 = Water Material 28 = Gravel # DENSM PORM # MAT 1 2250.0 0.339 2 2250 0 0 270 3 2250.0 1.000 4 1030 0 0 096 5 1030.0 0.185 2250 0 0 333 6 7 2250.0 0.288 2250.0 Q 1 000 9 1030.0 0 096 10 1030.0 0.182 11 2250.0 0.327 12 2250.0 0.280 13 2250.0 0.268 14 1030.0 1.000 15 1030.0 1.000 16 2250.0 0.323 17 2250.0 0.280 18 2250.0 0.257 19 1030.0 0.096 20 1030.0 1.000 21 2250.0 0.324

2250.0 0.467 22 23 2250.0 0 258 24 1030.0 1 000 1030 0 25 1 000 26 2529.4 0.150 27 1000.0 1.000 28 2700.0 0.300 Data on time series TINIT AGEO NTERM LREAD RAT TEND 0.00 0.50 1.25 1000.00 90 0 # Data on numerical parameters EPS EWT NLOOP MSPAR 1.00E-06 1.00E-18 3 1 Data on group (one set of data per group common for all sources) # Group No: 1 NNCH ISPEC TSPSOL 1 0 1 # Nuclide: C-14org TAU CSOLUB ATNV 5.7300E+03 1.35E+00 7.3428E-02 # CIRF % if ISPEC = 1 MAT DEFM SKDM % one line per material # 0.00315 0.000 1 2 0.00315 0.000 3 0.00315 0.000 0.06307 0 000 4 5 0.06307 0.000 6 0.00315 0.000 7 0.00315 0 000 0.00315 0 000 8 9 0.06307 0.000 10 0.06307 0.000 11 0.00315 0.000 12 0.00315 0.000 0.00315 13 0.000 0.06307 14 0.000 15 0.06307 0.000 16 0.00315 0.000 17 0.00315 0.000 18 0.00315 0.000 19 0.06307 0.000 0.06307 0.000 20 21 0.00315 0.000 0.00315 0.000 22 23 0.00315 0.000 2.4 0.06307 0.000 25 0.06307 0.000 26 0.00032 0.000 27 0.06307 0.000 28 0.01892 0.000 # Data on sources # NSBCHA ISEO MAT IFWATB VOLSOU FINVEN JINVEN # IBLOCK ISSEO % if ISEO = 1 # XXX % if JINVEN = 1. One line for each group.

```
#
# IBSOUR
                     % if ISEQ = 0
#
# IWINT
                       % if IFWATB = 1
# WBFLOW
                      % if IFWATB = 1 and IWINT = 0
# Waste
# Source 1
 1 0 1 0 7.9198E+02 1.0000000E+00 1
1.9681102E-01
1
# Source 2
1 0 1 0 1.9799E+02 1.000000E+00 1
#
4.9202755E-02
2
# Source 3
1 0 2 0 1.4311E+02 1.0000000E+00 1
#
2.4865513E-02
3
# Source 4
 1 0 2 0 3.5778E+01 1.0000000E+00 1
#
 6.2163783E-03
 4
# Source 5
 1 0 3 0 1.0000E-04 1.0000000E+00 1
#
 0.000000E+00
 5
# Source 6
 1 0 3 0 1.0000E-04 1.0000000E+00 1
#
0.000000E+00
 6
# Source 7
 1 0 4 1 2.0056E-05 1.0000000E+00 1
#
 3.6826632E-02
 7
 0
 2.00557103064067E-05
# Source 8
 1 0 4 1 1.2688E+02 1.0000000E+00 1
#
 0.000000E+00
 8
 0
2.00557103064067E-05
# Source 9
1 0 5 1 3.7759E-05 1.0000000E+00 1
#
 6.9333749E-02
 9
 0
 3.77590210448915E-05
# Source 10
 1 0 5 1 5.1890E+02 1.0000000E+00 1
0.000000E+00
 10
```

```
0
 3.77590210448915E-05
# Source 11
 1 0 6 0 1.0343E+03 1.0000000E+00 1
#
2.2628720E-01
 11
# Source 12
 1 0 6 0 2.5857E+02 1.000000E+00 1
#
 5.6571800E-02
 12
# Source 13
 1 0 7 0 7.3623E+01 1.0000000E+00 1
#
2.2236580E-03
13
# Source 14
 1 0 7 0 1.8406E+01 1.0000000E+00 1
#
5.5591451E-04
14
# Source 15
 1 0 8 0 1.0000E-04 1.0000000E+00 1
#
0.000000E+00
 15
# Source 16
 1 0 8 0 1.0000E-04 1.0000000E+00 1
#
0.000000E+00
 16
# Source 17
 1 0 9 1 5.2368E-05 1.0000000E+00 1
#
 9.6158429E-02
 17
 0
 5.23676880222841E-05
# Source 18
 1 0 9 1 3.3130E+02 1.0000000E+00 1
#
 0.000000E+00
 18
 0
 5.23676880222841E-05
# Source 19
 1 0 10 1 3.2264E-05 1.0000000E+00 1
#
5.9243025E-02
 19
 0
 3.22636330760221E-05
# Source 20
 1 0 10 1 4.2974E+02 1.0000000E+00 1
#
 0.000000E+00
 20
 0
 3.22636330760221E-05
# Source 21
 1 0 11 0 1.9303E+02 1.0000000E+00 1
#
 3.1367550E-02
```

```
21
# Source 22
 1 0 11 0 4.8259E+01 1.0000000E+00 1
#
7.8418875E-03
 22
# Source 23
 1 0 12 0 2.3745E+02 1.0000000E+00 1
1.6002049E-02
23
# Source 24
 1 0 12 0 5.9362E+01 1.0000000E+00 1
#
4.0005124E-03
24
# Source 25
 1 0 13 0 8.6889E+01 1.0000000E+00 1
#
0.000000E+00
25
# Source 26
1 0 13 0 2.1722E+01 1.0000000E+00 1
#
 0.000000E+00
 26
# Source 27
 1 0 14 0 1.0000E-04 1.0000000E+00 1
#
 0.000000E+00
 27
# Source 28
 1 0 14 0 1.0000E-04 1.0000000E+00 1
#
 0.000000E+00
 2.8
# Source 29
 1 0 15 0 1.0000E-04 1.0000000E+00 1
#
 0.000000E+00
 29
# Source 30
 1 0 15 0 1.0000E-04 1.0000000E+00 1
 0.000000E+00
 30
# Source 31
 1 0 16 0 3.2494E+01 1.0000000E+00 1
#
1.1228460E-03
 31
# Source 32
 1 0 16 0 8.1235E+00 1.0000000E+00 1
#
2.8071150E-04
 32
# Source 33
 1 0 17 0 6.6962E+01 1.0000000E+00 1
#
 4.8006148E-03
 33
# Source 34
1 0 17 0 1.6740E+01 1.0000000E+00 1
#
```

1.2001537E-03 34 # Source 35 1 0 18 0 6.4678E+00 1.0000000E+00 1 # 0.000000E+00 35 # Source 36 1 0 18 0 1.6169E+00 1.0000000E+00 1 # 0.000000E+00 36 # Source 37 1 0 19 1 5.0000E-05 1.0000000E+00 1 # 9.1810841E-02 37 0 0.00005 # Source 38 1 0 19 1 3.1632E+02 1.0000000E+00 1 # 0.000000E+00 38 0 0.00005 # Source 39 1 0 20 0 1.0000E-04 1.0000000E+00 1 # 0.000000E+00 39 # Source 40 1 0 20 0 1.0000E-04 1.0000000E+00 1 # 0.000000E+00 40 # Source 41 1 0 21 0 5.0936E+01 1.0000000E+00 1 # 2.6199740E-03 41 # Source 42 1 0 21 0 1.2734E+01 1.0000000E+00 1 # 6.5499349E-04 42 # Source 43 1 0 22 0 5.0967E+02 1.0000000E+00 1 # 1.1201435E-02 43 # Source 44 1 0 22 0 1.2742E+02 1.0000000E+00 1 # 2.8003587E-03 44 # Source 45 1 0 23 0 1.5817E+01 1.0000000E+00 1 # 0.000000E+00 45 # Source 46 1 0 23 0 3.9542E+00 1.0000000E+00 1 #

0.000000E+00 46 # Source 47 1 0 24 0 1.0000E-04 1.0000000E+00 1 0.000000E+00 47 # Source 48 1 0 24 0 1.0000E-04 1.0000000E+00 1 0.000000E+00 48 # Source 49 1 0 25 0 1.0000E-04 1.0000000E+00 1 0.000000E+00 49 # Source 50 1 0 25 0 1.0000E-04 1.0000000E+00 1 0.000000E+00 50 # Description of blocks not considered before # Block i # NZ NY NX NBLSP MAT IOFLOW # ZADIM YADIM XADIM VOLBLK % one line per compartment if NBLSP = 1 % if IOFLOW = 1 # IWINT # NODWIN NODWUT BFLOW % if IOFLOW = 1 and IWINT = 0 # Walls of concrete containers # Block 51 1 1 1 0 26 0 1.0000E+05 1.0774E-05 1.0774E-05 7.3875E+02 # Block 52 1 1 1 0 26 0 1.0000E+05 7.1788E-06 7.1788E-06 1.1087E+03 # Block 53 1 1 1 0 26 0 1.0000E+05 3.4498E-05 3.4498E-05 2.3071E+02 # Block 54 1 1 1 0 26 0 1.0000E+05 1.3526E-04 1.3526E-04 5.8843E+01 # Block 55 1 1 1 0 26 0 1.0000E+05 9.1075E-05 9.1075E-05 8.7391E+01 # Water inside structure # Block 56 1 1 1 0 27 1 1.7126E-07 3.4698E-07 9.2621E-07 6.5313E+02 1 # Block 57 1 1 1 0 27 1 1.3701E-07 2.7758E-07 9.2621E-07 5.9769E+02 1 # Block 58 1 1 1 0 27 1 3.4252E-07 6.9395E-07 9.2621E-07 4.4430E+02 1 # Block 59

1 1 1 0 27 1 6.8504E-07 1.3879E-06 9.2621E-07 8.3770E+01 1 # Block 60 1 1 1 0 27 1 4.5623E-07 9.2433E-07 9.2621E-07 3.1590E+02 1 # Short side of structure (passage) # Block 61 1 1 1 0 26 1 1.0000E+05 1.0000E+05 1.2349E-03 1.4396E+01 0 1 1 0.00257 # Block 62 1 1 1 0 26 1 1.0000E+05 1.0000E+05 1.2349E-03 1.4396E+01 0 1 1 0.00257 # Block 63 1 1 1 0 26 1 1.0000E+05 1.0000E+05 1.2349E-03 1.4396E+01 0 1 1 0.00257 # Ceiling of structure # Block 64 1 1 1 0 26 1 5.4232E-04 4.9206E-04 8.3616E+00 2.0379E+02 0 1 0.00644 1 # Block 65 1 1 1 0 26 1 5.4232E-04 4.9206E-04 8.3616E+00 2.0379E+02 0 1 1 0.00644 # Block 66 1 1 1 0 26 1 5.4232E-04 4.9206E-04 8.3616E+00 2.0379E+02 0 1 0.00644 1 # Block 67 1 1 1 0 26 1 4.3386E-04 3.9556E-04 1.0401E+01 2.5351E+02 0 1 1 0.0104 # Block 68 1 1 1 0 26 1 4.3386E-04 3.9556E-04 1.0401E+01 2.5351E+02 0 1 0.0104 1 # Block 69 1 1 1 0 26 1 4.3386E-04 3.9556E-04 1.0401E+01 2.5351E+02 0 1 1 0.0104 # Block 70 1 1 1 0 26 1 1.0846E-03 9.8891E-04 4.1606E+00 1.0140E+02 <u> </u>0 1 1 0.00596 # Block 71 1 1 1 0 26 1 1.0846E-03 9.8891E-04 4.1606E+00 1.0140E+02 0 1 1 0.00596

# Block 72 1 1 1 0 26 1 1.0846E-03 9.8891E-04 4.1606E+00 1.0140E+02 0 1 1 0.00596 # Block 73 1 1 1 0 26 1 2.1693E-03 1.9778E-03 2.0803E+00 5.0701E+01 0 1 1 0.01224 # Block 74 1 1 1 0 26 1 2.1693E-03 1.9778E-03 2.0803E+00 5.0701E+01 0 1 1 0.01224 # Block 75 1 1 1 0 26 1 2.1693E-03 1.9778E-03 2.0803E+00 5.0701E+01 0 1 0.01224 1 # Block 76 1 1 1 0 26 1 1.4447E-03 1.2679E-03 3.2450E+00 7.9088E+01 0 1 1 0.0049 # Block 77 1 1 1 0 26 1 1.4447E-03 1.2679E-03 3.2450E+00 7.9088E+01 0 1 0.0049 1 # Block 78 1 1 1 0 26 1 1.4447E-03 1.2679E-03 3.2450E+00 7.9088E+01 1 0 0 0049 1 # Bottom of structure # Block 79 1 1 1 0 26 1 4.2815E-04 3.8847E-04 1.0591E+01 1.6089E+02 0 1 1 0.00808 # Block 80 1 1 1 0 26 1 3.4252E-04 3.1229E-04 1.3175E+01 2.0014E+02 0 1 1 0.01228 # Block 81 1 1 1 0 26 1 8.5630E-04 7.8072E-04 5.2700E+00 8.0055E+01 0 1 1 0.00664 # Block 82 1 1 1 0 26 1 1.7126E-03 1.5614E-03 2.6350E+00 4.0027E+01 0 1 1 0.05001 # Block 83 1 1 1 0 26 1 1.1406E-03 1.0010E-03 4.1103E+00 6.2438E+01 0 1 0.03337 1 # Wall on left side of structure # Block 84 1 1 1 0 26 1

4.6264E-04 4.4246E-04 4.2411E+01 4.0179E+01 0 1 1 0.0018 # Block 85 1 1 1 0 26 1 4.6264E-04 4.4246E-04 4.2411E+01 4.0179E+01 0 1 1 0.0018 # Block 86 1 1 1 0 26 1 4.6264E-04 4.4246E-04 4.2411E+01 4.0179E+01 0 1 1 0.0018 # Block 87 1 1 1 0 26 1 3.7011E-04 3.5569E-04 5.2757E+01 4.9981E+01 0 1 1 0.00288 # Block 88 1 1 1 0 26 1 3.7011E-04 3.5569E-04 5.2757E+01 4.9981E+01 0 1 1 0.00288 # Block 89 1 1 1 0 26 1 3.7011E-04 3.5569E-04 5.2757E+01 4.9981E+01 0 1 1 0.00288 # Block 90 1 1 1 0 26 1 9.2527E-04 8.8923E-04 2.1103E+01 1.9992E+01 0 1 1 0.00131 # Block 91 1 1 1 0 26 1 9.2527E-04 8.8923E-04 2.1103E+01 1.9992E+01 0 1 0.00131 # Block 92 1 1 1 0 26 1 9.2527E-04 8.8923E-04 2.1103E+01 1.9992E+01 0 0.00131 # Block 93 1 1 1 0 26 1 1.8505E-03 1.7785E-03 1.0551E+01 9.9961E+00 0 1 1 0.00262 # Block 94 1 1 1 0 26 1 1.8505E-03 1.7785E-03 1.0551E+01 9.9961E+00 0 1 1 0.00262 # Block 95 1 1 1 0 26 1 1.8505E-03 1.7785E-03 1.0551E+01 9.9961E+00 0 1 1 0.00262 # Block 96 1 1 1 0 26 1 1.2324E-03 1.1401E-03 1.6459E+01 1.5593E+01 0 1 1 0.00141 # Block 97

1 1 1 0 26 1 1.2324E-03 1.1401E-03 1.6459E+01 1.5593E+01 0 1 1 0.00141 # Block 98 1 1 1 0 26 1 1.2324E-03 1.1401E-03 1.6459E+01 1.5593E+01 0 1 1 0.00141 # Wall on right side of structure # Block 99 1 1 1 0 26 1 4.6264E-04 4.4246E-04 4.2411E+01 4.0179E+01 0 1 1 0.00055 # Block 100 1 1 1 0 26 1 4.6264E-04 4.4246E-04 4.2411E+01 4.0179E+01 0 1 1 0.00055 # Block 101 1 1 1 0 26 1 4.6264E-04 4.4246E-04 4.2411E+01 4.0179E+01 0 1 1 0.00055 # Block 102 1 1 1 0 26 1 3.7011E-04 3.5569E-04 5.2757E+01 4.9981E+01 0 1 0.00068 1 # Block 103 1 1 1 0 26 1 3.7011E-04 3.5569E-04 5.2757E+01 4 9981E+01 0 1 1 0.00068 # Block 104 1 1 1 0 26 1 3.7011E-04 3.5569E-04 5.2757E+01 4.9981E+01 0 0 1 1 0.00068 # Block 105 1 1 1 0 26 1 9.2527E-04 8.8923E-04 2.1103E+01 1.9992E+01 0 1 1 0.00039 # Block 106 1 1 1 0 26 1 9.2527E-04 8.8923E-04 2.1103E+01 1.9992E+01 0 1 1 0.00039 # Block 107 1 1 1 0 26 1 9.2527E-04 8.8923E-04 2.1103E+01 1.9992E+01 0 1 0.00039 # Block 108 1 1 1 0 26 1 1.8505E-03 1.7785E-03 1.0551E+01 9.9961E+00 0 1 1 0.00265 # Block 109 1 1 1 0 26 1 1.8505E-03 1.7785E-03 1.0551E+01 9.9961E+00 0

1 1 0.00265 # Block 110 1 1 1 0 26 1 1.8505E-03 1.7785E-03 1.0551E+01 9.9961E+00 0 1 1 0.00265 # Block 111 1 1 1 0 26 1 1.2324E-03 1.1401E-03 1.6459E+01 1 5593E+01 0 1 1 0.00001 # Block 112 1 1 1 0 26 1 1.2324E-03 1.1401E-03 1.6459E+01 1 5593E+01 0 1 1 0.00001 # Block 113 1 1 1 0 26 1 1.2324E-03 1.1401E-03 1.6459E+01 1 5593E+01 0 1 1 0.00001 # Inner wall between room 4 and 5 # Block 114 1 1 1 0 26 1 1.0000E+05 1.0000E+05 3.7048E-03 4.3187E+01 0 1 1 0.00217 # Inner wall between room 9 and 10 # Block 115 1 1 1 0 26 1 1.0000E+05 1.0000E+05 3.7048E-03 4.3187E+01 0 1 1 0.00249 # Inner wall between room 11 and 12 # Block 116 1 1 1 0 26 1 1.0000E+05 1.0000E+05 3.7048E-03 4.3187E+01 0 1 1 0.0035 # Inner wall between room 12 and 13 # Block 117 1 1 1 0 26 1 1.0000E+05 1.0000E+05 3.7048E-03 4.3187E+01 0 1 0.029 # "Sink" 1 (inner wall between room 1 and 2, 2 and 3, 3 and 4) # Block 118 1 1 1 0 26 0 1.0000E+05 1.0000E+05 1.2349E-03 1.2956E+02 # "Sink" 2 (inner wall between room 5 and 6, 6 and 7, 7 and 8, 8 and 9) # Block 119 1 1 1 0 26 0 1.0000E+05 1.0000E+05 9.2621E-04 1.7275E+02 # "Sink" 3 (inner wall between room 10 and 11) # Block 120 1 1 1 0 26 0 1.0000E+05 1.0000E+05 3.7048E-03 4.3187E+01 # "Sink" 4 (no inner walls i.e. infinite small "sink") # Block 121 1 1 1 0 26 0 1.0000E+05 1.0000E+05 1.0000E+05 1.0000E-03 # "Sink" 5 (inner wall between room 13 and 14/15 14 and 15) # Block 122

1 1 1 0 26 0 1.0000E+05 1.0000E+05 3.7048E-03 4.3187E+01 # Short side of structure (loading zone) # Block 123 1 1 1 0 26 1 1.0000E+05 1.0000E+05 1.8524E-03 2.1593E+01 0 1 1 0.05607 # Block 124 1 1 1 0 26 1 1.0000E+05 1.0000E+05 1.8524E-03 2.1593E+01 0 1 1 0.05607 # Block 125 1 1 1 0 26 1 1.0000E+05 1.0000E+05 1.8524E-03 2.1593E+01 0 1 1 0.05607 # Gravel at the end of the tunnel (passage) # Block 126 1 1 1 0 28 1 1.5738E-02 6.1902E-03 2.1788E-03 2.0500E+02 # Block 127 1 1 1 0 28 1 1.0000E+05 6.1902E-03 2.1788E-03 2.0500E+02 # Block 128 1 1 1 0 28 1 1.0000E+05 6.1902E-03 2.1788E-03 2.0500E+02 # Gravel on top of the structure # Block 129 1 1 1 0 28 1 2.6779E-03 2.6178E-02 9.7206E-01 1.7530E+03 # Block 130 1 1 1 0 28 1 2.6779E-03 2.6178E-02 9.7206E-01 1.7530E+03 1 # Block 131 1 1 1 0 28 1 2.6779E-03 2.6178E-02 9.7206E-01 1.7530E+03 1 # Block 132 1 1 1 0 28 1 2.1528E-03 2.1044E-02 1.2092E+00 2.1807E+03 1 # Block 133 1 1 1 0 28 1 2.1528E-03 2.1044E-02 1.2092E+00 2.1807E+03 1 # Block 134 1 1 1 0 28 1 2.1528E-03 2.1044E-02 1.2092E+00 2.1807E+03 1 # Block 135 1 1 1 0 28 1 5.3819E-03 5.2611E-02 4.8367E-01 8.7227E+02 1 # Block 136 1 1 1 0 28 1 5.3819E-03 5.2611E-02 4.8367E-01 8.7227E+02 1

# Block 137 1 1 1 0 28 1 5.3819E-03 5.2611E-02 4.8367E-01 8.7227E+02 1 # Block 138 1 1 1 0 28 1 1.0764E-02 1.0522E-01 2.4184E-01 4.3613E+02 1 # Block 139 1 1 1 0 28 1 1.0764E-02 1.0522E-01 2.4184E-01 4.3613E+02 1 # Block 140 1 1 1 0 28 1 1.0764E-02 1.0522E-01 2.4184E-01 4.3613E+02 1 # Block 141 1 1 1 0 28 1 6.9004E-03 6.7455E-02 3.7724E-01 6.8032E+02 1 # Block 142 1 1 1 0 28 1 6.9004E-03 6.7455E-02 3.7724E-01 6.8032E+02 1 # Block 143 1 1 1 0 28 1 6.9004E-03 6.7455E-02 3.7724E-01 6.8032E+02 1 # Gravel in bottom of tunnel # Block 144 1 1 1 0 28 1 1.0100E-03 7.8534E-03 3.2402E+00 5.2591E+02 1 # Block 145 1 1 1 0 28 1 8.1195E-04 6.3133E-03 4.0306E+00 6.5420E+02 1 # Block 146 1 1 1 0 28 1 2.0299E-03 1.5783E-02 1.6122E+00 2.6168E+02 1 # Block 147 1 1 1 0 28 1 4.0597E-03 3.1567E-02 8.0612E-01 1.3084E+02 1 # Block 148 1 1 1 0 28 1 2.6026E-03 2.0237E-02 1.2575E+00 2.0409E+02 1 # Gravel on left side of structure # Block 149 1 1 1 0 28 1 1.0270E-01 6.6535E-03 2.4222E+00 7.0351E+02 1 # Block 150 1 1 1 0 28 1 8.2559E-02 5.3487E-03 3.0131E+00 8.7513E+02 1 # Block 151 1 1 1 0 28 1 2.0640E-01 1.3372E-02 1.2052E+00 3.5005E+02 1 # Block 152 1 1 1 0 28 1

4.1279E-01 2.6744E-02 6.0261E-01 1.7503E+02 1 # Block 153 1 1 1 0 28 1 2.6463E-01 1.7145E-02 9.4000E-01 2.7302E+02 1 # Gravel on right side of structure # Block 154 1 1 1 0 28 1 1.0270E-01 6.6535E-03 2.4222E+00 7.0351E+02 1 # Block 155 1 1 1 0 28 1 8.2559E-02 5.3487E-03 3.0131E+00 8.7513E+02 1 # Block 156 1 1 1 0 28 1 2.0640E-01 1.3372E-02 1.2052E+00 3.5005E+02 1 # Block 157 1 1 1 0 28 1 4.1279E-01 2.6744E-02 6.0261E-01 1.7503E+02 1 # Block 158 1 1 1 0 28 1 2.6463E-01 1.7145E-02 9.4000E-01 2.7302E+02 # Gravel at the end of the tunnel (loading zone) # Block 159 1 1 1 0 28 1 1.4549E-01 5.7224E-02 2.0142E-02 1.8951E+03 1 # Block 160 1 1 1 0 28 1 1.0000E+05 5.7224E-02 1.8951E+03 2.0142E-02 1 # Block 161 1 1 1 0 28 1 1.0000E+05 5.7224E-02 2.0142E-02 1.8951E+03 1 # Codifying the connections # NSSCO NSBCO NBBCO 25 25 166 # Connection between sources # NABCO ICSEQ LSCFW IRADD # QWATIN % if LSCFW is not equal to 0 # ISERNU % if IRADD = 1 # XRADD % if IRADD = 1 and ISERNU = 0 one line per group # IBSA IBSB ISEQA ISEQB % if NABCO > 1 and ICSEQ = 1 # IBSA IBSB % if ICSEQ = 0 NABCO\*lines # WITHIN WASTE 1 Source-source 1 - 2 # 0 1 0 1 1 1.185245E-02

# TBSOU TBSB ISEOC NABCCO TRZB ICRB IPLUG TRADD TCOMPB ICSEOB % if NABCCO > 1 and ISEOC =1 # # TDCB % if ISEOC = 0. One value for each connection. LSCFW OWATIN % if IFWATB(source term) = 1 and IOFLOW(block) = 1 # ISERNU % if IRADD =1 # XRADD % if IRADD = 1 and ISERNU = 1 # XRADD % if IRADD = 1 and ISERNU = 0. One line per group. # BETWEEN WASTE - CONCRETE WALLS OF CONCRETE CONTAINERS # 1 Source-block 2 - 51 2 51 0 1 1 1 0 1 1 # 1 7.90732583E-04 # 2 Source-block 12 - 52 12 52 0 1 1 1 0 1 1 1 5.26878043E-04 # 3 Source-block 22 - 53 22 53 0 1 1 1 0 1 1 # 1 2 53194171E-03 # 4 Source-block 32 - 54 32 54 0 1 1 1 0 1 1 1 9.92721699E-03 # 5 Source-block 42 - 55 42 55 0 1 1 1 0 1 1 # 1 6.68432610E-03 # BETWEEN WASTE - WATER # 6 Source-block 4 - 56 4 56 0 1 0 0 0 1 1 # 1 4.74751081E-03 # 7 Source-block 6 - 56 6 56 0 1 0 0 0 1 1 #

```
1
1.0000000E+12
# 8 Source-block 8 - 56
   8 56 0 1 0 0 0 1
 1
 1 2.00557103064067E-05
 1
 2.94730830E-04
# 9 Source-block 10 - 56
   10 56 0 1 0 0 0 1
 1
 1 3.77590210448915E-05
 1
2.56014226E-05
# 10 Source-block 14 - 57
  14 57 0 1 0 0 0 1
#
1
#
1
 7.20311984E-03
# 11 Source-block 16 - 57
   16 57 0 1 0 0 0 1
 1
#
 1
 1.00000000E+12
# 12 Source-block 18 - 57
  18 57 0 1 0 0 0 1
#
 1
          5.23676880222841E-05
 1
 1
 1 12875637E-04
# 13 Source-block 20 - 57
  20 57 0 1 0 0 0 1
 1
 1
          3.22636330760221E-05
 1
 3.74272453E-05
# 14 Source-block 24 - 58
   24 58 0 1 0 0 0 1
1
#
 1
2.38731972E-03
# 15 Source-block 26 - 58
  26 58 0 1 0 0 0 1
#
1
#
 1
 1.09619884E-03
# 16 Source-block 28 - 58
   28 58 0 1 0 0 0 1
#
 1
#
 1
```

```
1.0000000E+12
# 17 Source-block 30 - 58
   30 58 0 1 0 0 0 1
 1
#
1
1.0000000E+12
# 18 Source-block 34 - 59
   34 59 0 1 0 0 0 1
 1
#
1
8.52614186E-03
# 19 Source-block 36 - 59
  36 59 0 1 0 0 0 1
 1
#
1
2.47253739E-02
# 20 Source-block 38 - 59
  38 59 0 1 0 0 0 1
 1
 1
          0.00005
 1
 1.18220723E-04
# 21 Source-block 40 - 59
   40 59 0 1 0 0 0 1
 1
#
 1
1.0000000E+12
# 22 Source-block 44 - 60
  44 60 0 1 0 0 0 1
 1
#
 1
 1.49207482E-03
# 23 Source-block 46 - 60
   46 60 0 1 0 0 0 1
 1
#
1
9.94540179E-03
# 24 Source-block 48 - 60
  48 60 0 1 0 0 0 1
1
#
 1
 1.0000000E+12
# 25 Source-block 50 - 60
   50 60 0 1 0 0 0 1
 1
#
 1
 1.0000000E+12
```

```
# Connections between blocks not considered as sources
# TBA TBB ISEOC NABCCO IRZA IRZB ICRA ICRB IPLUG IRADD
# TDCA TDCB
                           % if NABCCO = 1
# ICOMPA ICOMPB ICSEOA ICSEOB % if NABCCO > 1 and ISOC = 1
                           % if NABCCO > 1 and ISOC = 0
# TDCA
# TDCB
                           % if NABCCO > 1 and ISQC = 0
# LSCFW OWATIN
                    % if IOFLOW(A) = 1 and IOFLOW(B) = 1
#
# ISERNII
                    % if IRADD = 1
#
# YRADD
        % if IRADD = 1 and ISERNU = 0. One line per group
#
# BETWEEN WALLS OF CONCRETE CONTAINERS AND WATER
# 1 Block-block 51 - 56
  51 56 0 1 2 0 1 0 0 0
  1 1
# 2 Block-block 52 - 57
  52 57 0 1 2 0 1 0 0 0
  1
          1
# 3 Block-block 53 - 58
  53 58 0 1 2 0 1 0 0 0
  1 1
# 4 Block-block 54 - 59
  54 59 0 1 2 0 1 0 0 0
  1 1
# 5 Block-block 55 - 60
  55 60 0 1 2 0 1 0 0 0
  1 1
# BETWEEN WATER AND SHORT SIDE OF THE STRUCTURE (PASSAGE)
# 6 Block-block 56 - 61
  56 61 0 1 2 2 0 1 0 0
  1 1
  1 2.57000E-03
# BETWEEN WATER AND CEILING OF THE STRUCTURE
# 7 Block-block 56 - 64
  56 64 0 1 0 0 0 1 0 0
 1 1
1 6.44000E-03
# 8 Block-block 57 - 67
  57 67 0 1 0 0 0 1 0 0
 1 1
1 1.04000E-02
# 9 Block-block 58 - 70
  58 70 0 1 0 0 0 1 0 0
  1 1
1 5.96000E-03
# 10 Block-block 59 - 73
  59 73 0 1 0 0 0 1 0 0
  1 1
1 1.22400E-02
# 11 Block-block 60 - 76
  60 76 0 1 0 0 0 1 0 0
     1
4.90000E-03
  1
  1
# BETWEEN WATER AND BOTTOM OF THE STRUCTURE
# 12 Block-block 56 - 79
  56 79 0 1 0 0 0 1 0 0
  1 1
```

```
8.08000E-03
  - 1
# 13 Block-block 57 - 80
 57 80 0 1 0 0 0 1 0 0
  1
         1
  - 1
       1.22800E-02
# 14 Block-block 58 - 81
  58 81 0 1 0 0 0 1 0 0
  1
         1
  -1 6.64000E-03
# 15 Block-block 59 - 82
  59 82 0 1 0 0 0 1 0 0
  1
         1
  -1 5.00100E-02
# 16 Block-block 60 - 83
 60 83 0 1 0 0 0 1 0 0
  1
         1
  -1
       3.33700E-02
# BETWEEN WATER AND WALL ON THE LEFT SIDE OF THE STRUCTURE
# 17 Block-block 56 - 84
 56 84 0 1 1 0 0 1 0 0
  1
         1
  1 1.80000E-03
# 18 Block-block 57 - 87
 57 87 0 1 1 0 0 1 0 0
  1
         1
  1
        2.88000E-03
# 19 Block-block 58 - 90
 58 90 0 1 1 0 0 1 0 0
  1
         1
  1
       1.31000E-03
# 20 Block-block 59 - 93
 59 93 0 1 1 0 0 1 0 0
         1
  1
  1
       2.62000E-03
# 21 Block-block 60 - 96
 60 96 0 1 1 0 0 1 0 0
         1
  1
  1
       1.41000E-03
# BETWEEN WATER AND WALL ON THE RIGHT SIDE OF THE STRUCTURE
# 22 Block-block 56 - 99
  56 99 0 1 1 0 0 1 0 0
  1 1
-1 5.50000E-04
# 23 Block-block 57 - 102
  57 102 0 1 1 0 0 1 0 0
  1 1
  -1 6.80000E-04
# 24 Block-block 58 - 105
 58 105 0 1 1 0 0 1 0 0
  1 1
        3.90000E-04
  1
# 25 Block-block 59 - 108
 59 108 0 1 1 0 0 1 0 0
  1 1
  1
        2.65000E-03
# 26 Block-block 60 - 111
  60 111 0 1 1 0 0 1 0 0
  1 1
  - 1
         1.00000E-05
# BETWEEN WATER AND WALLS BETWEEN ROOM 4 AND 5 IN THE STRUCTURE
# 27 Block-block 56 - 114
  56 114 0 1 2 2 0 1 0 0
  1 1
  -1 2.17000E-03
```

```
# 28 Block-block 57 - 114
  57 114 0 1 2 2 0 1 0 0
         1
  1
  1
          2.17000E-03
# BETWEEN WATER AND WALLS BETWEEN ROOM 9 AND 10 IN THE STRUCTURE
# 29 Block-block 57 - 115
  57 115 0 1 2 2 0 1 0 0
  1
          1
  -1 2.49000E-03
# 30 Block-block 58 - 115
  58 115 0 1 2 2 0 1 0 0
  1
          1
  1
       2.49000E-03
# BETWEEN WATER AND WALLS BETWEEN ROOM 11 AND 12 IN THE STRUCTURE
# 31 Block-block 58 - 116
  58 116 0 1 2 2 0 1 0 0
  1
       1
  - 1
         3.50000E-03
# 32 Block-block 59 - 116
  59 116 0 1 2 2 0 1 0 0
  1 1
  1
        3.50000E-03
# BETWEEN WATER AND WALLS BETWEEN ROOM 12 AND 13 IN THE STRUCTURE
# 33 Block-block 59 - 117
  59 117 0 1 2 2 0 1 0 0
  1
           1
  1
        2.90000E-02
# 34 Block-block 60 - 117
  60 117 0 1 2 2 0 1 0 0
  1
           1
  -1
          2.90000E-02
# BETWEEN WATER AND WALLS INSIDE ROOMS 1-4 IN THE STRUCTURE
# 35 Block-block 56 - 118
  56 118 0 1 2 2 0 1 0 0
  1 1
# BETWEEN WATER AND WALLS INSIDE ROOMS 5-9 IN THE STRUCTURE
# 36 Block-block 57 - 119
  57 119 0 1 2 2 0 1 0 0
  1 1
# BETWEEN WATER AND WALLS INSIDE ROOMS 10-11 IN THE STRUCTURE
# 37 Block-block 58 - 120
  58 120 0 1 2 2 0 1 0 0
  1 1
# BETWEEN WATER AND WALLS INSIDE ROOM 12 IN THE STRUCTURE
# 38 Block-block 59 - 121
  59 121 0 1 2 2 0 1 0 0
  1 1
# BETWEEN WATER AND WALLS INSIDE ROOMS 13-15 IN THE STRUCTURE
# 39 Block-block 60 - 122
  60 122 0 1 2 2 0 1 0 0
  1 1
# BETWEEN WATER AND SHORT SIDE OF THE STRUCTURE (LOADING ZONE)
# 40 Block-block 60 - 123
  60 123 0 1 2 2 0 1 0 0
  1 1
1 5.60700E-02
# WITHIN SHORT SIDE OF THE STRUCTURE (PASSAGE)
# 41 Block-block 61 - 62
  61 62 0 1 2 2 1 1 0 0
  1 1
1 2.57000E-03
# 42 Block-block 62 - 63
  62 63 0 1 2 2 1 1 0 0
```

```
1
          1
  1 2.57000E-03
# BETWEEN SHORT SIDE OF THE STRUCTURE (PASSAGE)
# AND GRAVEL AT THE SHORT SIDE OF THE STRUCTURE (PASSAGE)
# 43 Block-block 63 - 126
  63 126 0 1 2 1 1 1 0 0
  1
          1
  1 2.57000E-03
# WITHIN CEILING OF THE STRUCTURE (TRANSVERSE TO THE MAIN AXIS OF
TUNNEL)
# 44 Block-block 64 - 65
  64 65 0 1 1 1 1 1 0 0
  1
         1
  1 6.44000E-03
# 45 Block-block 65 - 66
  65 66 0 1 1 1 1 1 0 0
  1
          1
  1 6.44000E-03
# 46 Block-block 67 - 68
  67 68 0 1 1 1 1 1 0 0
  1
          1
        1.04000E-02
  1
# 47 Block-block 68 - 69
  68 69 0 1 1 1 1 1 0 0
  1
          1
  1
         1.04000E-02
# 48 Block-block 70 - 71
  70 71 0 1 1 1 1 1 0 0
  1
          1
  1
         5.96000E-03
# 49 Block-block 71 - 72
  71 72 0 1 1 1 1 1 0 0
          1
  1
  1
        5.96000E-03
# 50 Block-block 73 - 74
  73 74 0 1 1 1 1 1 0 0
          1
  1
  1
        1.22400E-02
# 51 Block-block 74 - 75
  74 75 0 1 1 1 1 1 0 0
  1
          1
  1
        1.22400E-02
# 52 Block-block 76 - 77
  76 77 0 1 1 1 1 1 0 0
  1 1
  1 4.90000E-03
# 53 Block-block 77 - 78
  77 78 0 1 1 1 1 1 0 0
  1
        1
        4.90000E-03
   1
# WITHIN CEILING OF THE STRUCTURE (ALONG THE MAIN AXIS OF TUNNEL)
# 54 Block-block 64 - 67
# 55 Block-block 65 - 68
# 56 Block-block 66 - 69
# 57 Block-block 67 - 70
# 58 Block-block 68 - 71
# 59 Block-block 69 - 72
# 60 Block-block 70 - 73
# 61 Block-block 71 - 74
# 62 Block-block 72 - 75
# 63 Block-block 73 - 76
# 64 Block-block 74 - 77
# 65 Block-block 75 - 78
# BETWEEN CEILING OF THE STRUCTURE AND GRAVEL ON TOP OF THE STRUCTURE
```

```
# 66 Block-block 66 - 129
 66 129 0 1 1 0 1 1 0 0
  1 1
  1 6.44000E-03
# 67 Block-block 69 - 132
  69 132 0 1 1 0 1 1 0 0
  1 1
 1 1.04000E-02
# 68 Block-block 72 - 135
  72 135 0 1 1 0 1 1 0 0
  1
         1
  1 5.96000E-03
# 69 Block-block 75 - 138
  75 138 0 1 1 0 1 1 0 0
  1 1
  1 1.22400E-02
# 70 Block-block 78 - 141
  78 141 0 1 1 0 1 1 0 0
  1 1
  1
       4.90000E-03
# WITHIN BOTTOM OF THE STRUCTURE
# 71 Block-block 79 - 80
# 72 Block-block 80 - 81
# 73 Block-block 81 - 82
# 74 Block-block 82 - 83
# BETWEEN BOTTOM OF THE STRUCTURE AND GRAVEL IN BOTTOM OF THE
STRUCTURE
# 75 Block-block 79 - 144
  79 144 0 1 1 0 1 1 0 0
  1 1
-1 8.08000E-03
# 76 Block-block 80 - 145
  80 145 0 1 1 0 1 1 0 0
  1 1
-1 1.22800E-02
# 77 Block-block 81 - 146
  81 146 0 1 1 0 1 1 0 0
  1 1
-1 6.64000E-03
# 78 Block-block 82 - 147
  82 147 0 1 1 0 1 1 0 0
  1 1
-1 5.00100E-02
# 79 Block-block 83 - 148
  83 148 0 1 1 0 1 1 0 0
  1 1
-1 3.33700E-02
# WITHIN LEFT WALL OF THE STRUCTURE (TRANSVERSE TO THE MAIN AXIS OF
TUNNEL)
# 80 Block-block 84 - 85
  84 85 0 1 1 1 1 1 0 0
 1 1
1 1.80000E-03
# 81 Block-block 85 - 86
  85 86 0 1 1 1 1 1 0 0
  1 1
1 1.80000E-03
# 82 Block-block 87 - 88
  87 88 0 1 1 1 1 1 0 0
  1 1
1 2.88000E-03
# 83 Block-block 88 - 89
 88 89 0 1 1 1 1 1 0 0
  1 1
```

```
1 2.88000E-03
# 84 Block-block 90 - 91
  90 91 0 1 1 1 1 1 0 0
        1
  1
  1 1.31000E-03
# 85 Block-block 91 - 92
  91 92 0 1 1 1 1 1 0 0
  1 1
  1 1.31000E-03
# 86 Block-block 93 - 94
  93 94 0 1 1 1 1 1 0 0
  1 1
  1 2.62000E-03
# 87 Block-block 94 - 95
 94 95 0 1 1 1 1 1 0 0
  1 1
       2.62000E-03
  1
# 88 Block-block 96 - 97
  96 97 0 1 1 1 1 1 0 0
  1 1
  1 1.41000E-03
# 89 Block-block 97 - 98
 97 98 0 1 1 1 1 1 0 0
  1 1
  1
         1.41000E-03
# WITHIN LEFT WALL OF THE STRUCTURE (ALONG THE MAIN AXIS OF TUNNEL)
# 90 Block-block 84 - 87
# 91 Block-block 85 - 88
# 92 Block-block 86 - 89
# 93 Block-block 87 - 90
# 94 Block-block 88 - 91
# 95 Block-block 89 - 92
# 96 Block-block 90 - 93
# 97 Block-block 91 - 94
# 98 Block-block 92 - 95
# 99 Block-block 93 - 96
# 100 Block-block 94 - 97
# 101 Block-block 95 - 98
# BETWEEN LEFT WALL OF THE STRUCTURE AND GRAVEL ON LEFT SIDE OF THE
STRUCTURE
# 102 Block-block 86 - 149
  86 149 0 1 1 1 1 1 0 0
  1 1
  1 1.80000E-03
# 103 Block-block 89 - 150
  89 150 0 1 1 1 1 1 0 0
  1 1
1 2.88000E-03
# 104 Block-block 92 - 151
  92 151 0 1 1 1 1 1 0 0
  1 1
        1.31000E-03
  1
# 105 Block-block 95 - 152
  95 152 0 1 1 1 1 1 0 0
  1 1
  1
        2.62000E-03
# 106 Block-block 98 - 153
  98 153 0 1 1 1 1 1 0 0
  1
          1
  1
         1.41000E-03
# WITHIN RIGHT WALL OF THE STRUCTURE (TRANSVERSE TO THE MAIN AXIS OF
TUNNEL)
# 107 Block-block 99 - 100
  99 100 0 1 1 1 1 1 0 0
```

```
1
          1
  -1 5.50000E-04
# 108 Block-block 100 - 101
  100 101 0 1 1 1 1 1 0 0
  1
         1
  -1 5.50000E-04
# 109 Block-block 102 - 103
  102 103 0 1 1 1 1 1 0 0
         1
  1
  -1 6.80000E-04
# 110 Block-block 103 - 104
  103 104 0 1 1 1 1 1 0 0
  1
         1
  -1
        6.80000E-04
# 111 Block-block 105 - 106
  105 106 0 1 1 1 1 1 0 0
  1 1
       3.90000E-04
  1
# 112 Block-block 106 - 107
  106 107 0 1 1 1 1 1 0 0
  1
         1
       3.90000E-04
  1
# 113 Block-block 108 - 109
  108 109 0 1 1 1 1 1 0 0
  1
          1
  1
        2.65000E-03
# 114 Block-block 109 - 110
  109 110 0 1 1 1 1 1 0 0
  1
          1
  1
         2.65000E-03
# 115 Block-block 111 - 112
 111 112 0 1 1 1 1 1 0 0
          1
  1
  - 1
        1.00000E-05
# 116 Block-block 112 - 113
 112 113 0 1 1 1 1 1 0 0
          1
  1
  - 1
         1.00000E-05
# WITHIN RIGHT WALL OF THE STRUCTURE (ALONG THE MAIN AXIS OF TUNNEL)
# 117 Block-block 99 - 102
# 118 Block-block 100 - 103
# 119 Block-block 101 - 104
# 120 Block-block 102 - 105
# 121 Block-block 103 - 106
# 122 Block-block 104 - 107
# 123 Block-block 105 - 108
# 124 Block-block 106 - 109
# 125 Block-block 107 - 110
# 126 Block-block 108 - 111
# 127 Block-block 109 - 112
# 128 Block-block 110 - 113
# BETWEEN RIGHT WALL OF THE STRUCTURE AND GRAVEL ON RIGHT SIDE OF THE
STRUCTURE
# 129 Block-block 101 - 154
  101 154 0 1 1 1 1 1 0 0
  1 1
  -1
          5.50000E-04
# 130 Block-block 104 - 155
  104 155 0 1 1 1 1 1 0 0
  1 1
-1 6.80000E-04
# 131 Block-block 107 - 156
 107 156 0 1 1 1 1 1 0 0
  1 1
```

```
1 3.90000E-04
# 132 Block-block 110 - 157
  110 157 0 1 1 1 1 1 0 0
  1
          1
  1 2.65000E-03
# 133 Block-block 113 - 158
  113 158 0 1 1 1 1 1 0 0
  1
          1
   -1 1.00000E-05
# WITHIN SHORT SIDE OF THE STRUCTURE (LOADING ZONE)
# 134 Block-block 123 - 124
  123 124 0 1 2 2 1 1 0 0
  1 1
  1 5.60700E-02
# 135 Block-block 124 - 125
  124 125 0 1 2 2 1 1 0 0
  1 1
  1 5.60700E-02
# BETWEEN SHORT SIDE OF THE STRUCTURE (LOADING ZONE) AND
# GRAVEL AT THE SHORT SIDE OF THE STRUCTURE (LOADING ZONE)
# 136 Block-block 125 - 159
  125 159 0 1 2 1 1 1 0 0
  1
          1
  1
         5.60700E-02
# WITHIN GRAVEL AT THE SHORT SIDE OF THE STRUCTURE (LOADING ZONE)
# 137 Block-block 126 - 127
  126 127 0 1 2 2 1 1 0 0
  1 1
  1
        3.37910E-01
# 138 Block-block 127 - 128
  127 128 0 1 2 2 1 1 0 0
  1
      1
         2.75740E-01
  1
# BETWEEN GRAVEL AT THE SHORT SIDE OF THE STRUCTURE (PASSAGE)
# AND GRAVEL ON TOP OF STRUCTURE
# 139 Block-block 126 - 129
  126 129 0 1 0 2 1 1 0 0
  1 1
  -1 4.69933E-02
# 140 Block-block 126 - 130
  126 130 0 1 0 2 1 1 0 0
  1 1
  -1 4.69933E-02
# 141 Block-block 126 - 131
  126 131 0 1 0 2 1 1 0 0
  1 1
  -1 4.69933E-02
# BETWEEN GRAVEL AT THE SHORT SIDE OF THE STRUCTURE (PASSAGE)
# AND GRAVEL IN BOTTOM OF STRUCTURE
# 142 Block-block 126 - 144
  126 144 0 1 0 2 1 1 0 0
  1 1
-1 1.48180E-01
# BETWEEN GRAVEL AT THE SHORT SIDE OF THE STRUCTURE (PASSAGE)
# AND GRAVEL ON LEFT SIDE OF STRUCTURE
# 143 Block-block 126 - 149
  126 149 0 1 0 2 1 1 0 0
  1 1
-1 4.18900E-02
# BETWEEN GRAVEL AT THE SHORT SIDE OF THE STRUCTURE (PASSAGE)
# AND GRAVEL ON RIGHT SIDE OF STRUCTURE
# 144 Block-block 126 - 154
  126 154 0 1 0 2 1 1 0 0
  1 1
```

```
- 1
        6 64600E-02
# WITHIN GRAVEL ON TOP OF THE STRUCTURE (TRANSVERSE TO THE MAIN AXIS
OF TUNNEL)
# 145 Block-block 129 - 130
 129 130 0 1 0 0 1 1 0 0
 1 1
 1 1.10247E+00
# 146 Block-block 130 - 131
 130 131 0 1 0 0 1 1 0 0
 1 1
 1 1.21367E+00
# 147 Block-block 132 - 133
 132 133 0 1 0 0 1 1 0 0
 1 1
 1 1.51314E+00
# 148 Block-block 133 - 134
 133 134 0 1 0 0 1 1 0 0
 1 1
      1.53118E+00
 1
# 149 Block-block 135 - 136
 135 136 0 1 0 0 1 1 0 0
  1 1
       8.12880E-01
  1
# 150 Block-block 136 - 137
 136 137 0 1 0 0 1 1 0 0
  1 1
  1
       7.39800E-01
# 151 Block-block 138 - 139
 138 139 0 1 0 0 1 1 0 0
  1 1
       1.43958E+00
  1
# 152 Block-block 139 - 140
 139 140 0 1 0 0 1 1 0 0
 1 1
  1
       1.15865E+00
# 153 Block-block 141 - 142
 141 142 0 1 0 0 1 1 0 0
  1 1
 1
       6.94280E-01
# 154 Block-block 142 - 143
 142 143 0 1 0 0 1 1 0 0
  1 1
1 6.21860E-01
# WITHIN GRAVEL ON TOP OF THE STRUCTURE (ALONG THE MAIN AXIS OF
TUNNEL)
# 155 Block-block 129 - 132
 129 132 0 1 2 2 1 1 0 0
 1 1
  -1
        2.65263E-01
# 156 Block-block 130 - 133
 130 133 0 1 2 2 1 1 0 0
  1 1
  - 1
        2.65263E-01
# 157 Block-block 131 - 134
 131 134 0 1 2 2 1 1 0 0
  1 1
  -1
         2.65263E-01
# 158 Block-block 132 - 135
 132 135 0 1 2 2 1 1 0 0
  1 1
-1 4.05833E-01
# 159 Block-block 133 - 136
 133 136 0 1 2 2 1 1 0 0
  1 1
```

```
4.49320E-01
  - 1
# 175 Block-block 138 - 157
 138 157 0 1 1 0 1 1 0 0
  1
         1
  -1 7.44810E-01
# 176 Block-block 141 - 158
 141 158 0 1 1 0 1 1 0 0
  1
         1
 -1 4.69360E-01
# WITHIN GRAVEL IN BOTTOM OF THE STRUCTURE (ALONG THE MAIN AXIS OF
TUNNEL)
# 177 Block-block 144 - 145
 144 145 0 1 2 2 1 1 0 0
  1 1
  -1 1.72350E-01
# 178 Block-block 145 - 146
 145 146 0 1 2 2 1 1 0 0
  1
         1
  -1 3.55690E-01
# 179 Block-block 146 - 147
 146 147 0 1 2 2 1 1 0 0
  1 1
  -1 6.64060E-01
# 180 Block-block 147 - 148
 147 148 0 1 2 2 1 1 0 0
  1
         1
       7.55000E-03
  1
# BETWEEN GRAVEL IN BOTTOM OF THE STRUCTURE AND GRAVEL ON LEFT SIDE OF
THE STRUCTURE
# 181 Block-block 144 - 149
 144 149 0 1 1 0 1 1 0 0
  1 1
  1
       4.55150E-01
# 182 Block-block 145 - 150
 145 150 0 1 1 0 1 1 0 0
  1 1
  1
       6 66360E-01
# 183 Block-block 146 - 151
  146 151 0 1 1 0 1 1 0 0
  1 1
       3.98290E-01
  1
# 184 Block-block 147 - 152
  147 152 0 1 1 0 1 1 0 0
  1 1
1 1.46270E+00
  1
# 185 Block-block 148 - 153
 148 153 0 1 1 0 1 1 0 0
  1 1
1 3.62560E-01
# BETWEEN GRAVEL IN BOTTOM OF THE STRUCTURE AND GRAVEL ON RIGHT SIDE
OF THE STRUCTURE
# 186 Block-block 144 - 154
  144 154 0 1 1 0 1 1 0 0
  1 1
1 4.22130E-01
# 187 Block-block 145 - 155
  145 155 0 1 1 0 1 1 0 0
  1 1
1 6.04260E-01
# 188 Block-block 146 - 156
 146 156 0 1 1 0 1 1 0 0
 1 1
1 3.63810E-01
# 189 Block-block 147 - 157
```

```
1
             1
  1
          1.29990E-01
# BETWEEN GRAVEL AT THE SHORT SIDE OF THE STRUCTURE (LOADING ZONE)
# AND GRAVEL IN BOTTOM OF STRUCTURE
# 204 Block-block 159 - 148
  159 148 0 1 0 2 1 1 0 0
  1
            1
  -1
            1.02140E+00
# BETWEEN GRAVEL AT THE SHORT SIDE OF THE STRUCTURE (LOADING ZONE)
# AND GRAVEL ON LEFT SIDE OF STRUCTURE
# 205 Block-block 159 - 153
  159 153 0 1 0 2 1 1 0 0
  1
            1
  -1
            9.88600E-02
# BETWEEN GRAVEL AT THE SHORT SIDE OF THE STRUCTURE (LOADING ZONE)
# AND GRAVEL ON RIGHT SIDE OF STRUCTURE
# 206 Block-block 159 - 158
  159 158 0 1 0 2 1 1 0 0
  1
            1
            2.35590E-01
   - 1
```

# 1BTF CALCULATIONS Problem Codification: BS2 # Nuclide(s): C-14 org # Problem Description: PROPER VERSION INPUT DATA FORMAT # # ITYPE NGROUP NSINKS IWFLOW NSOURC NBSIZE ISPSOL # 1 1 0 1 50 0 0 # Flow stream conditions # NINFLW NUTFLW % if TWFLOW = 1 36 11 Inflow conditions (one line per inflow) # # OINFW NBWIN NBNWIN % if IWFLOW = 1 0.056706666666666 126 1 127 0.056706666666666 1 0.120536666666667 128 1 0.0629053333333333 129 1 0.0629053333333333 130 1 0.062905333333333 131 1 132 0.5178703333333333 1 0.5178703333333333 133 1 0.5178703333333333 134 1 0.510755333333334 135 1 0.5107633333333333 136 1 0.5107633333333333 137 1 0.02928633333333333 138 1 0.02928633333333333 139 1 0.02928633333333333 140 1 0.03152933333333333 141 1 0.03152933333333333 142 1 0.03152933333333333 143 1 0.488112 144 1 0.603498 145 1 0.469099 146 1 0.218313 147 1 0.413202 148 1 0.116626666666667 159 1 0.116626666666667 160 1 0.129276666666667 161 1 0.091684 163 1 0.131073 165 1 0.027931 167 1 0.028474 169 1 0.052161 171 1 0.067122 173 1 0.020592 175 1 0.144678 177 1 0.042487 179 1 0.054792 181 1 # Outflow conditions (one line per inflow) # QUTFW NBWOUT NBNWOUT % if IWFLOW = 1 126 0.00324 1 127 0.00324 1 128 0.00324 1 0.189676 131 1 3.695231 134 1 2.35931 137 1 0.097219 140 1 0.295508 143 1

0.0866966666666667 159 1 0.086696666666666 160 1 0.0866966666666667 161 1 # Number of blocks connections and materials # NBLOCK NBCOUP NMAT 289 9 195 # Data on materials (one line per material) # Material 1 = Waste tankar # Material 2 = Construction concrete # Material 3 = Water # Material 4 = Gravel # Material 5 = Porous concrete # Material 6 = Waste drums # Material 7 = Waste "Berglofslador" # Material 8 = Concrete moulds including waste # Material 9 = Cement between inner and outer drums # MAT DENSM PORM 2250.0 1 0.633 2 2529.4 0.150 1000.0 3 1.000 2700.0 4 0.300 5 2625.0 0.200 6 2250.0 0.095 7 2250.0 0.098 8 2250.0 0.248 9 2250.0 0.200 Data on time series # # TINIT AGEO RAT TEND NTERM LREAD 0.00 0.50 1.23 1000.00 90 0 # # Data on numerical parameters # EPS EWT NLOOP MSPAR 1.00E-06 1.00E-18 3 1 # Data on group (one set of data per group common for all sources) # # Group No: 1 # NNCH ISPEC ISPSOL 1 1 0 # Nuclide: C-14 org # TAU CSOLUB AINV 5.7300E+03 1.00E+10 7.7519E-02 # # CIRF % if ISPEC = 1 # MAT DEFM % one line per material # SKDM 0.06307 0.000 1 2 0.00032 0.000 0.06307 0.000 3 4 0.01892 0.000 5 0.00315 0.000 6 0.06307 0.000 7 0.06307 0.000 0.00032 0.000 8 9 0.00315 0.000 Data on sources # # NSBCHA ISEO MAT IFWATB VOLSOU FINVEN JINVEN # # IBLOCK ISSEQ % if ISEQ = 1

XXX % if JINVEN = 1. One line for each group. # TBSOUR % if ISEO = 0 # IWINT % if IFWATB = 1 # WBFLOW % if IFWATB = 1 and IWINT = 0 Waste # Source 1 1 0 1 0 0.0010 1.000000E+00 1 # 0.000000E+00 1 # Source 2 1 0 1 1 336.0000 1.000000E+00 1 # 1.0174918E-02 2 0 5.591650E-01 # Source 3 1 0 1 0 0.0010 1.000000E+00 1 0.000000E+00 3 # Source 4 1 0 1 0 0.0010 1.0000000E+00 1 # 0.000000E+00 4 # Source 5 1 0 1 0 0.0010 1.000000E+00 1 # 0.000000E+00 5 # Source 6 1 0 6 1 680.2950 1.000000E+00 1 # 0.000000E+00 6 0 5.591650E-01 # Source 7 1 0 1 0 0.0010 1.000000E+00 1 # 0.000000E+00 7 # Source 8 1 0 1 0 0.0010 1.000000E+00 1 # 0.000000E+00 8 # Source 9 1 0 1 0 0.0010 1.000000E+00 1 # 0.000000E+00 9 # Source 10 1 0 1 0 0.0010 1.000000E+00 1 0.000000E+00 10
```
# Source 11
 1 0 1 0 0.0010 1.0000000E+00 1
#
 0.000000E+00
11
# Source 12
1 0 1 1 570.0000 1.000000E+00 1
#
 4.1078860E-03
 12
 0
 5.858700E-01
# Source 13
 1 0 1 0 0.0010 1.0000000E+00 1
#
0.000000E+00
13
# Source 14
1 0 1 0 0.0010 1.000000E+00 1
#
0.000000E+00
14
# Source 15
 1 0 1 0 0.0010 1.000000E+00 1
#
0.000000E+00
 15
# Source 16
 1 0 1 0 0.0010 1.000000E+00 1
#
 0.000000E+00
 16
# Source 17
 1 0 7 1 4.9211 1.0000000E+00 1
#
 8.5549378E-02
 17
 0
 5.858700E-01
# Source 18
 1 0 7 1 4.9211 1.000000E+00 1
#
 8.5549378E-02
 18
 0
 5.858700E-01
# Source 19
 1 0 1 0 0.0010 1.000000E+00 1
#
 0.000000E+00
 19
# Source 20
 1 0 1 0 0.0010 1.000000E+00 1
#
0.000000E+00
 20
# Source 21
 1 0 1 0 0.0010 1.000000E+00 1
#
 0.000000E+00
 21
# Source 22
1 0 1 1 570.0000 1.000000E+00 1
#
```

```
4.1078860E-03
 22
 0
 6.387300E-01
# Source 23
 1 0 1 0 0.0010 1.000000E+00 1
#
0.000000E+00
 23
# Source 24
 1 0 1 0 0.0010 1.000000E+00 1
#
0.000000E+00
# 24
# Source 25
1 0 1 0 0.0010 1.0000000E+00 1
#
0.000000E+00
 25
# Source 26
 1 0 1 0 0.0010 1.0000000E+00 1
#
0.000000E+00
 26
# Source 27
 1 0 7 1 4.9211 1.000000E+00 1
#
 8.5549378E-02
 27
 0
 6.387300E-01
# Source 28
 1 0 7 1 4.9211 1.000000E+00 1
#
 8.5549378E-02
 28
 0
 6.387300E-01
# Source 29
 1 0 1 0 0.0010 1.0000000E+00 1
#
0.000000E+00
29
# Source 30
 1 0 1 0 0.0010 1.000000E+00 1
#
0.000000E+00
 30
# Source 31
 1 0 1 0 0.0010 1.0000000E+00 1
#
0.000000E+00
31
# Source 32
 1 0 1 1 612.0000 1.000000E+00 1
#
4.4502098E-03
 32
 0
 5.420300E-01
# Source 33
1 0 1 0 0.0010 1.000000E+00 1
#
```

```
0.000000E+00
 33
# Source 34
 1 0 1 0 0.0010 1.000000E+00 1
#
0.000000E+00
34
# Source 35
 1 0 1 0 0.0010 1.0000000E+00 1
 0.000000E+00
35
# Source 36
 1 0 1 0 0.0010 1.0000000E+00 1
#
0.000000E+00
36
# Source 37
1 0 7 1 5.2105 1.000000E+00 1
#
 9.0581694E-02
 37
 0
 5.420300E-01
# Source 38
 1 0 7 1 5.2105 1.000000E+00 1
 9.0581694E-02
 38
 0
 5.420300E-01
# Source 39
 1 0 1 0 0.0010 1.000000E+00 1
#
 0.000000E+00
 39
# Source 40
 1 0 1 0 0.0010 1.000000E+00 1
 0.000000E+00
 40
# Source 41
 1 0 1 0 0.0010 1.000000E+00 1
 0.000000E+00
 41
# Source 42
 1 0 1 1 1518.0000 1.0000000E+00 1
#
 1.0954363E-02
 42
 0
 7.311550E-01
# Source 43
 1 0 1 0 0.0010 1.000000E+00 1
#
 0.000000E+00
 43
# Source 44
 1 0 1 0 0.0010 1.000000E+00 1
 0.000000E+00
 44
# Source 45
```

```
1 0 1 0 0.0010 1.000000E+00 1
#
 0.000000E+00
 45
# Source 46
 1 0 1 0 0.0010 1.0000000E+00 1
0.000000E+00
 46
# Source 47
 1 0 7 1 12.7368 1.0000000E+00 1
 2.2142192E-01
 47
 0
 7.311550E-01
# Source 48
 1 0 7 1 12.7368 1.0000000E+00 1
#
 2.2142192E-01
 48
 0
 7.311550E-01
# Source 49
 1 0 1 0 0.0010 1.000000E+00 1
#
0.000000E+00
 49
# Source 50
 1 0 1 0 0.0010 1.000000E+00 1
#
 0.000000E+00
 50
# Description of blocks not considered before
# Block i
# NZ NY NX NBLSP MAT IQFLOW
# ZADIM YADIM XADIM VOLBLK % one line per compartment if NBLSP = 1
                       % if IOFLOW = 1
# IWINT
# NODWIN NODWUT BFLOW % if IOFLOW = 1 and IWINT = 0
# Walls of concrete tanks 51-55 and 118-122
# Block 51
 1 1 1 0 2 1
 1.0000E+05 2.4351E-04 1.8013E-04
                                      1.1200E+02
 0
          1 0.559165
 1
#
# Block 52
 1 1 1 0 2 1
 1.0000E+05 1.4354E-04 1.0618E-04
                                      1.9000E+02
 0
 1
            1 0.58587
#
# Block 53
 1 1 1 0 2 1
 1.0000E+05 1.4354E-04 1.0618E-04
                                      1.9000E+02
 0
 1
       1 0.63873
#
# Block 54
```

```
1 1 1 0 2 1
1.0000E+05 1.3369E-04 9.8896E-05 2.0400E+02
 0
1
     1 0.54203
#
# Block 55
1 1 1 0 2 1
1.0000E+05 5.3899E-05 3.9871E-05 5.0600E+02
0
1 1 0.731155
# Kringgjutningsbetong
# Block 56
1 1 1 0 5 1
4.3447E-06 1.4402E-04 3.7185E-05 5.5072E+02
1
#
# Block 57
1 1 1 0 5 1
2.1793E-04 6.0585E-04
                   3.6026E-05
                              2.0819E+02
1
# Block 58
1 1 1 0 5 1
2.1793E-04 6.0585E-04 3.6026E-05
                              2.0819E+02
1
#
# Block 59
1 1 1 0 5 1
2.0117E-04 5.5925E-04 3.3553E-05 2.2554E+02
1
# Block 60
1 1 1 0 5 1
 8.1725E-05 2.2719E-04 1.3527E-05
                              5.5519E+02
# Short side of structure (passage)
# Block 61
1 1 1 0 2 1
1.0000E+05 3.3307E-03 3.1010E-03 2.0155E+01
         1 0.0812
1
# Block 62
1 1 1 0 2 1
1.0000E+05 3.1010E-03 3.1010E-03
                              2.0155E+01
0
    1 0.0812
1
# Fiktivt block 63
# Block 63
1 1 1 0 3 1
1.0000E-05 1.0000E-05 1.0000E-05
                              1.0000E-02
0
   1 0.0812
1
# Ceiling
# Block 64
1 1 1 0 2 1
 2.4244E+00 2.2051E-04 2.2308E-04
                              7.9693E+01
 0
     1 0.47797
1
# Block 65
1 1 1 0 2 1
2.4244E+00 2.2051E-04 1.0000E+05
                              7.9693E+01
 0
```

1 1 0.47797 # Block 66 1 1 1 0 2 1 2.4244E+00 2.2051E-04 1.0000E+05 7.9693E+01 0 1 1 0.47797 # # Block 67 1 1 1 0 2 1 3.1027E+00 1.1419E-03 1.1419E-03 6.2272E+01 0 1 1 0.44204 # # Block 68 1 1 1 0 2 1 3.1027E+00 1.1419E-03 1.0000E+05 6.2272E+01 0 1 1 0.44204 # Block 69 1 1 1 0 2 1 3.1027E+00 1.1419E-03 1.0000E+05 6.2272E+01 0 1 0.44204 1 # # Block 70 1 1 1 0 2 1 3.1027E+00 1.1419E-03 1.1419E-03 6.2272E+01 0 1 0.36708 1 # # Block 71 1 1 1 0 2 1 3.1027E+00 1.1419E-03 1.0000E+05 6.2272E+01 0 1 1 0.36708 # # Block 72 1 1 1 0 2 1 3.1027E+00 1.1419E-03 1.0000E+05 6.2272E+01 0 1 0.36708 1 # Block 73 1 1 1 0 2 1 2.8640E+00 1.0541E-03 1.0541E-03 6.7461E+01 0 1 1 0.24072 # # Block 74 1 1 1 0 2 1 2.8640E+00 1.0541E-03 1.0000E+05 6.7461E+01 0 1 1 0.24072 # # Block 75 1 1 1 0 2 1 2.8640E+00 1.0541E-03 1.0000E+05 6.7461E+01 0 1 0.24072 1 # Block 76 1 1 1 0 2 1

```
1.1635E+00 4.2350E-04 4.2823E-04
                                1.6606E+02
 0
      1 0.36594
 1
#
# Block 77
 1 1 1 0 2 1
 1.1635E+00 4.2350E-04 1.0000E+05
                                1.6606E+02
 0
 1 1 0.36594
# Block 78
 1 1 1 0 2 1
 1.1635E+00 4.2350E-04 1.0000E+05
                                1.6606E+02
 0
          1
                 0.36594
1
# Bottom of structure 79-83 and 182-191
# Block 79
 1 1 1 0 2 1
 2.2989E-02 2.0851E-04 2.2308E-04
                                8.5260E+01
1
#
# Block 80
1 1 1 0 2 1
 5.9524E-02 5.3990E-04
                    5.7097E-04
                                3.2928E+01
 1
# Block 81
 1 1 1 0 2 1
 5.9524E-02 5.3990E-04
                     5.7097E-04
                                3.2928E+01
 1
# Block 82
 1 1 1 0 2 1
 5.4945E-02 4.9837E-04 5.2705E-04
                                 3.5672E+01
 1
#
# Block 83
 1 1 1 0 2 1
 2.2075E-02 2.0023E-04
                    2.1411E-04
                                 8.8788E+01
 1
# Fictitious block 84-113
# Block 84
 1 1 1 0 3 1
 1.0000E-05 1.0000E+05 1.0000E-05
                                 1.0000E-02
 0
      1 0.08172
 1
# Block 85
 1 1 1 0 3 1
 1.0000E-05 1.0000E+05 1.0000E-05
                                1.0000E-02
 0
      1 0.08172
 1
# Block 86
 1 1 1 0 3 1
 1.0000E-05 1.0000E+05 1.0000E-05
                                1.0000E-02
 0
      1 0.08172
 1
# Block 87
 1 1 1 0 3 1
 1.0000E-05 1.0000E+05 1.0000E-05
                                1.0000E-02
 0
 1 1 0.04921
```

# Block 88 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 0.04921 1 # Block 89 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.04921 # Block 90 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.02749 # Block 91 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.02749 # Block 92 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 0.02749 1 # Block 93 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.02734 # # Block 94 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 0.02734 1 # Block 95 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.02734 # # Block 96 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.04747 # Block 97 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.04747 # Block 98 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02

```
0
1 1 0.04747
# Fictitious block (Wall on right side of structure) 99-113
# Block 99
1 1 1 0 3 1
1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02
1
#
# Block 100
1 1 1 0 3 1
1.0000E-05 1.0000E+05
                     1.0000E-05
                                 1.0000E-02
1
#
# Block 101
1 1 1 0 3 1
1.0000E-05 1.0000E+05 1.0000E-05
                                 1.0000E-02
1
#
# Block 102
1 1 1 0 3 1
1.0000E-05 1.0000E+05
                      1.0000E-05
                                 1.0000E-02
1
#
# Block 103
1 1 1 0 3 1
 1.0000E-05 1.0000E+05
                     1.0000E-05
                                   1.0000E-02
1
#
# Block 104
1 1 1 0 3 1
 1.0000E-05 1.0000E+05
                     1.0000E-05
                                 1.0000E-02
1
#
# Block 105
1 1 1 0 3 1
 1.0000E-05 1.0000E+05 1.0000E-05
                                 1.0000E-02
1
#
# Block 106
1 1 1 0 3 1
 1.0000E-05 1.0000E+05 1.0000E-05
                                 1.0000E-02
1
# Block 107
 1 1 1 0 3 1
 1.0000E-05 1.0000E+05 1.0000E-05
                                 1.0000E-02
1
#
# Block 108
1 1 1 0 3 1
1.0000E-05 1.0000E+05 1.0000E-05
                                 1.0000E-02
1
#
# Block 109
1 1 1 0 3 1
1.0000E-05 1.0000E+05
                     1.0000E-05
                                  1.0000E-02
1
# Block 110
1 1 1 0 3 1
1.0000E-05 1.0000E+05
                     1.0000E-05
                                 1.0000E-02
1
# Block 111
```

1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # # Block 112 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Block 113 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Fictiotious block 114-117 # Block 114 1 1 1 0 3 1 1.0000E+05 1.0000E+05 1.0000E+05 1.0000E-02 0 1 1 0.00536 # # Block 115 1 1 1 0 3 1 1.0000E+05 1.0000E+05 1.0000E+05 1.0000E-02 0 1 0.13847 1 # Block 116 1 1 1 0 3 1 1.0000E+05 1.0000E+05 1.0000E+05 1.0000E-02 0 1 0.27165 1 # # Block 117 1 1 1 0 3 1 1.0000E+05 1.0000E+05 1.0000E+05 1.0000E-02 1.... 0 1 0.30131 # Walls of conrete tanks 118-122 # Block 118 1 1 1 0 2 1 1.0000E+05 2.4351E-04 1.8013E-04 1.1200E+02 0 1 0.559165 1 # Block 119 1 1 1 0 2 1 1.0000E+05 1.4354E-04 1.0618E-04 1.9000E+02 0 1 0.58587 1 # # Block 120 1 1 1 0 2 1 1.0000E+05 1.4354E-04 1.0618E-04 1.9000E+02 0 1 0.63873 1 # # Block 121 1 1 1 0 2 1 1.0000E+05 1.3369E-04 9.8896E-05 2.0400E+02 0 1 1 0.54203 # Block 122 1 1 1 0 2 1

```
1.0000E+05 5.3899E-05 3.9871E-05
                                  5.0600E+02
 0
 1
          1 0.731155
# Short side of structure (loading zone)
# Block 123
1 1 1 0 2 1
1.0000E+05 3.5971E-03 3.1010E-03
                                  2.0155E+01
 0
      1 0.36522
1
#
# Block 124
1 1 1 0 2 1
1.0000E+05 3.1010E-03 3.1010E-03
                                 2.0155E+01
 0
1
          1
                   0.36522
# Fictitious block 125
# Block 125
1 1 1 0 3 1
 1.0000E+05 1.0000E+05
                     1.0000E+05
                                  1.0000E-02
 0
1
          1
                   0.36522
# Gravel at the end of the tunnel (passage) 126-128
# Block 126
1 1 1 0 4 1
 1.4506E-01 1.6538E-02 1.0829E-02 1.6417E+02
 1
# Block 127
1 1 1 0 4 1
 1.0000E+05 1.0000E+05 1.0829E-02
                                 1.6417E+02
 1
#
# Block 128
1 1 1 0 4 1
1.0000E+05 1.0000E+05
                     1.0829E-02 1.6417E+02
# Gravel on top of the structure 129-143
# Block 129
 1 1 1 0 4 1
 9.7785E-04 1.0781E-01 4.7325E+00
                                 3.9984E+02
 1
# Block 130
 1 1 1 0 4 1
 9.7785E-04 1.0000E+05 4.7325E+00 3.9984E+02
1
# Block 131
1 1 1 0 4 1
 9.7785E-04 1.0000E+05 4.7325E+00
                                 3.9984E+02
1
#
# Block 132
 1 1 1 0 4 1
 2.5319E-03 2.7914E-01 1.8277E+00 1.5442E+02
1
# Block 133
1 1 1 0 4 1
 2.5319E-03 1.0000E+05
                      1.8277E+00
                                  1.5442E+02
 1
#
# Block 134
1 1 1 0 4 1
```

2.5319E-03 1.0000E+05 1.8277E+00 1.5442E+02 1 # # Block 135 1 1 1 0 4 1 2.5319E-03 2.7914E-01 1.8277E+00 1.5442E+02 1 # # Block 136 1 1 1 0 4 1 2.5319E-03 1.0000E+05 1.8277E+00 1.5442E+02 1 # # Block 137 1 1 1 0 4 1 2.5319E-03 1.0000E+05 1.8277E+00 1.5442E+02 1 # # Block 138 1 1 1 0 4 1 2.3372E-03 2.5767E-01 1.9801E+00 1.6729E+02 1 # Block 139 1 1 1 0 4 1 2.3372E-03 1.0000E+05 1.9801E+00 1.6729E+02 1 # # Block 140 1 1 1 0 4 1 2.3372E-03 1.0000E+05 1.9801E+00 1.6729E+02 1 # # Block 141 1 1 1 0 4 1 9.3899E-04 1.0352E-01 4.9284E+00 4.1638E+02 1 # # Block 142 1 1 1 0 4 1 9.3899E-04 1.0000E+05 4.9284E+00 4.1638E+02 1 # Block 143 1 1 1 0 4 1 9.3899E-04 1.0000E+05 4.9284E+00 4.1638E+02 1 # Gravel in bottom of tunnel 144-148 # Block 144 1 1 1 0 4 1 4.6915E-04 1.0000E+05 9.7506E+00 1.9184E+02 1 # # Block 145 1 1 1 0 4 1 1.2148E-03 1.0000E+05 3.8095E+00 7.4088E+01 1 # Block 146 1 1 1 0 4 1 1.2148E-03 1.0000E+05 3.8095E+00 7.4088E+01 1 # Block 147

```
1 1 1 0 4 1
1.1213E-03 1.0000E+05 4.1270E+00 8.0262E+01
1
#
# Block 148
1 1 1 0 4 1
4.5051E-04 1.0000E+05
                       1.0159E+01
                                   1.9977E+02
1
# Concrete on left side 149-153 and 162-171
# Block 149
1 1 1 0 5 1
1.0000E+00 5.2847E-04 5.5603E+01
                                   3.3640E+01
1
#
# Block 150
1 1 1 0 5 1
2.5893E+00 1.3684E-03
                       2.1724E+01
                                   1.2992E+01
1
#
# Block 151
1 1 1 0 5 1
2.5893E+00 1.3684E-03
                      2.1724E+01
                                    1.2992E+01
1
# Block 152
1 1 1 0 5 1
 2.3901E+00 1.2631E-03
                       2.3534E+01
                                   1.4075E+01
 1
# Block 153
1 1 1 0 5 1
 9.6026E-01 5.0747E-04 5.7931E+01
                                   3.5032E+01
# Concrete on right side 154-158 and 172-181
# Block 154
1 1 1 0 5 1
 1.0000E+00 5.2847E-04 5.5603E+01
                                  3.3640E+01
1
# Block 155
1 1 1 0 5 1
 2.5893E+00 1.3684E-03 2.1724E+01
                                   1.2992E+01
1
# Block 156
1 1 1 0 5 1
 2.5893E+00 1.3684E-03
                      2.1724E+01
                                  1.2992E+01
1
# Block 157
1 1 1 0 5 1
 2.3901E+00 1.2631E-03
                       2.3534E+01
                                    1.4075E+01
1
#
# Block 158
1 1 1 0 5 1
9.6026E-01 5.0747E-04
                      5.7931E+01
                                   3.5032E+01
1
# Gravel at the end of the tunnel (loading zone) 159-161
# Block 159
1 1 1 0 4 1
 5.5848E-01 6.3673E-02 4.1692E-02 6.3204E+02
 1
#
```

# Block 160 1 1 1 0 4 1 1.0000E+05 1.0000E+05 4.1692E-02 6.3204E+02 1 # Block 161 1 1 1 0 4 1 1.0000E+05 1.0000E+05 4.1692E-02 6.3204E+02 1 # Concrete on left side 149-153 and 162-171 # Block 162 1 1 1 0 5 1 1.0000E+05 5.2847E-04 5.5603E+01 3.3640E+01 1 # # Block 163 1 1 1 0 5 1 1.0000E+05 5.2847E-04 5.5603E+01 3.3640E+01 1 # # Block 164 1 1 1 0 5 1 1.0000E+05 1.3684E-03 2.1724E+01 1.2992E+01 1 # Block 165 1 1 1 0 5 1 1.0000E+05 1.3684E-03 2.1724E+01 1.2992E+01 1 # # Block 166 1 1 1 0 5 1 1.0000E+05 1.3684E-03 1.2992E+01 2.1724E+01 1 # # Block 167 1 1 1 0 5 1 1.0000E+05 1.3684E-03 2.1724E+01 1.2992E+01 1 # # Block 168 1 1 1 0 5 1 1.0000E+05 1.2631E-03 2.3534E+01 1.4075E+01 1 # # Block 169 1 1 1 0 5 1 1.0000E+05 1.2631E-03 2.3534E+01 1.4075E+01 1 # # Block 170 1 1 1 0 5 1 1.0000E+05 5.0747E-04 5.7931E+01 3.5032E+01 1 # # Block 171 1 1 1 0 5 1 1.0000E+05 5.0747E-04 5.7931E+01 3.5032E+01 1 # Concrete on right side 154-158 and 172-181 # Block 172 1 1 1 0 5 1 1.0000E+05 5.2847E-04 5.5603E+01 3.3640E+01 1

```
# Block 173
1 1 1 0 5 1
1.0000E+05 5.2847E-04 5.5603E+01 3.3640E+01
1
#
# Block 174
1 1 1 0 5 1
1.0000E+05 1.3684E-03
                    2.1724E+01
                                1.2992E+01
1
# Block 175
1 1 1 0 5 1
1.0000E+05 1.3684E-03 2.1724E+01
                               1 2992E+01
1
#
# Block 176
1 1 1 0 5 1
1.0000E+05 1.3684E-03
                     2.1724E+01
                               1.2992E+01
1
#
# Block 177
1 1 1 0 5 1
1.0000E+05 1.3684E-03 2.1724E+01
                                1.2992E+01
1
#
# Block 178
1 1 1 0 5 1
1.0000E+05 1.2631E-03 2.3534E+01
                               1.4075E+01
1
# Block 179
1 1 1 0 5 1
 1.0000E+05 1.2631E-03 2.3534E+01
                               1.4075E+01
1
#
# Block 180
1 1 1 0 5 1
1.0000E+05 5.0747E-04 5.7931E+01 3.5032E+01
1
# Block 181
1 1 1 0 5 1
1.0000E+05 5.0747E-04 5.7931E+01
                               3.5032E+01
1
# Bottom of structure 79-83 and 182-191
# Block 182
1 1 1 0 2 1
1.1494E-02 2.0851E-04 2.2194E+01
                               8.5260E+01
 0
      1 0.46813
1
#
# Block 183
1 1 1 0 2 1
1.1494E-02 2.0851E-04 2.2194E+01
                               8.5260E+01
0
1
      1 0.46813
# Block 184
1 1 1 0 2 1
 2.9762E-02 5.3990E-04 8.5714E+00
                                3.2928E+01
 0
    1 0.570663
1
```

# Block 185 1 1 1 0 2 1 2.9762E-02 5.3990E-04 8.5714E+00 3 2928E+01 0 1 1 0.570663 # Block 186 1 1 1 0 2 1 2.9762E-02 5.3990E-04 8.5714E+00 3.2928E+01 0 1 1 0.458716 # Block 187 1 1 1 0 2 1 2.9762E-02 5.3990E-04 8.5714E+00 3.2928E+01 0 1 1 0.458716 # Block 188 1 1 1 0 2 1 2.7473E-02 4.9837E-04 9.2857E+00 3.5672E+01 0 1 1 0.232102 # Block 189 1 1 1 0 2 1 2.7473E-02 4.9837E-04 9.2857E+00 3.5672E+01 0 1 0.232102 1 # Block 190 1 1 1 0 2 1 1.1038E-02 2.0023E-04 2.3112E+01 8.8788E+01 0 1 0.380536 1 # # Block 191 1 1 1 0 2 1 1.1038E-02 2.0246E-04 2.3112E+01 8.8788E+01 0 1 0.380536 1 # Cement between inner and outer drums # Block 192 1 1 1 0 9 1 1.0000E+05 9.8219E-06 1.6006E-05 3.9321E+02 0 1 0.559165 1 # # Block 193 1 1 1 0 9 1 1.0000E+05 9.8219E-06 1.6006E-05 3.9321E+02 0 7 1 0.559165 # Walls of concrete moulds # Block 194 1 1 1 0 8 1 1.0000E+05 1.0000E+05 6.4300E-04 1.8662E+02 0 1 0.559165 1 # Block 195 1 1 1 0 8 1 1.0000E+05 1.0000E+05 6.4300E-04 1.8662E+02 0

```
1
                           0.559165
 1
  Codifying the connections
#
  NSSCO
          NSBCO
                   NBBCO
#
                   265
  4
          20
#
  Connection between sources
  NABCO ICSEO
                  LSCFW
                         IRADD
#
#
  QWATIN
                    % if LSCFW is not equal to 0
#
  ISERNU
                   % if IRADD = 1
#
  XRADD
                   % if IRADD = 1 and ISERNU = 1 one line per group
#
#
# XRADD
                   % if IRADD = 1 and ISERNU = 0 one line per group
# IBSA IBSB ISEOA ISEOB
                             % if NABCO > 1 and ICSEO = 1
#
 IBSA IBSB
                              % if ICSEO = 0 NABCO*lines
#
#
  WITHIN WASTE
#
#
  1 Source-source 17 - 18
    1
                   0
                                  1
                                                 1
    5.858700E-01
    1
    1.000000E-03
    17
                    18
#
  2 Source-source 27 - 28
                   0
                                  1
                                                 1
    1
    6.387300E-01
    1
    1.000000E-03
    27
                    28
# 3 Source-source 37 - 38
    1
                   0
                                  1
                                                 1
    5.420300E-01
    1
    1.000000E-03
    37
                    38
  4 Source-source 47 - 48
    1
                   0
                                  1
                                                 1
    7.311550E-01
    1
    1.000000E-03
    47
                   48
  Connection source - block
#
  IBSOU
         IBSB
                 ISEOC
                         NABCCO
                                  IRZB
                                          ICRB
                                                  IPLUG
                                                           IRADD
  ICOMPB ICSEOB % if NABCCO > 1 and ISEOC =1
#
#
  IDCB
                  % if ISEOC = 0. One value for each connection.
#
  LSCFW OWATIN
                 % if IFWATB(source term) = 1 and IQFLOW(block) = 1
# ISERNU
                  % if IRADD =1
```

```
XRADD
                 % if IRADD = 1 and ISERNU = 1
#
                 % if IRADD = 1 and ISERNU = 0. One line per group.
#
  XRADD
  BETWEEN WASTE - CONCRETE WALLS OF CONCRETE TANKS
#
  1 Source-block 2 - 51
    2 51 0 1 1 1 0 0
 1
 1 0.559165
# 2 Source-block 2 - 118
    2 118 0 1 1 1 0 0
#
 1
 -1 0.559165
# BETWEEN WASTE - CEMENT BETWEEN DRUM IN DRUM
# 3 Source-block 6 - 192
    6 192 0 1 2 1 0 0
#
 1
 1 0.559165
# 4 Source-block 6 - 193
    6 193 0 1 2 1 0 0
 1
 -1 0.559165
# 5 Source-block 12 - 52
    12 52 0 1 1 1 0 0
#
 1
 1 0.58587
# 6 Source-block 12 - 119
    12 119 0 1 1 1 0 0
 1
 -1 0.58587
# BETWEEN WASTE IN 'BERGLOFSLADOR' - CEMENT AROUND WASTE PACKAGES
# 7 Source-block 17 - 57
    17 57 0 1 1 1 0 0
 1
 -1 0.58587
# 8 Source-block 18 - 57
    18 57 0 1 1 1 0 0
#
 1
 1 0.58587
# 9 Source-block 22 - 53
    22 53 0 1 1 1 0 0
 1
 1 0.63873
# 10 Source-block 22 - 120
    22 120 0 1 1 1 0 0
 1
 -1 0.63873
# 11 Source-block 27 - 58
    27 58 0 1 1 1 0 0
#
 1
 -1 0.63873
```

```
# 12 Source-block 28 - 58
    28 58 0 1 1 1 0 0
#
 1
 1 0.63873
# 13 Source-block 32 - 54
   32 54 0 1 1 1 0 0
 1
 1 0.54203
# 14 Source-block 32 - 121
   32 121 0 1 1 1 0 0
#
1
 -1 0.54203
# 15 Source-block 37 - 59
   37 59 0 1 1 1 0 0
#
 1
 -1 0.54203
# 16 Source-block 38 - 59
   38 59 0 1 1 1 0 0
#
1
 1 0.54203
# 17 Source-block 42 - 55
   42 55 0 1 1 1 0 0
#
 1
 1 0.731155
# 18 Source-block 42 - 122
   42 122 0 1 1 1 0 0
#
 1
 -1 0.731155
# 19 Source-block 47 - 60
   47 60 0 1 1 1 0 0
 1
 -1 0.731155
# 20 Source-block 48 - 60
   48 60 0 1 1 1 0 0
 1
 1 0 731155
# Connections between blocks not considered as sources
#
# IBA IBB ISEQC NABCCO IRZA IRZB ICRA ICRB IPLUG IRADD
#
# IDCA IDCB
                            % if NABCCO = 1
#
# ICOMPA ICOMPB ICSEOA ICSEOB % if NABCCO > 1 and ISOC = 1
# IDCA
                            % if NABCCO > 1 and ISOC = 0
# IDCB
                            \% if NABCCO > 1 and ISOC = 0
# LSCFW OWATIN
                            % if IOFLOW(A) = 1 and IOFLOW(B) = 1
# ISERNU
                             % if IRADD = 1
# XRADD
                  % if IRADD = 1 and ISERNU = 0. One line per group
# BETWEEN WALLS OF CONCRETE TANKS AND BACKFILL CONCRETE
```

```
# 1 Block-block 51 - 56
  51 56 0 1 2 2 1 1 0 0
         1
  1
  1 5.59165E-01
# 2 Block-block 52 - 57
  52 57 0 1 2 2 1 1 0 0
  1
         1
  1
         5.85870E-01
# 3 Block-block 53 - 58
  53 58 0 1 2 2 1 1 0 0
  1
         1
  1
         6.38730E-01
# 4 Block-block 54 - 59
  54 59 0 1 2 2 1 1 0 0
  1
         1
  1 5.42030E-01
# 5 Block-block 55 - 60
  55 60 0 1 2 2 1 1 0 0
  1
         1
        7.31155E-01
  1
# 6 Block-block 56 - 118
  56 118 0 1 2 2 1 1 0 0
  1
          1
  1
         5.59165E-01
# 7 Block-block 57 - 119
  57 119 0 1 2 2 1 1 0 0
  1
          1
  1
        5.85870E-01
# 8 Block-block 58 - 120
  58 120 0 1 2 2 1 1 0 0
  1
          1
  1
        6.38730E-01
# 9 Block-block 59 - 121
  59 121 0 1 2 2 1 1 0 0
  1
         1
  1
        5.42030E-01
# 10 Block-block 60 - 122
  60 122 0 1 2 2 1 1 0 0
         1
  1
        7.31155E-01
  1
# BETWEEN CONCRETE BACKFILL AND WALL BETWEEN WASTE AND GRAVEL
BACKFILL (PASSAGE)
# 11 Block-block 56 - 61
  56 61 0 1 0 1 1 1 0 0
  1 1
  1
       8.12000E-02
# BETWEEN CONCRETE BACKFILL AND CONCRETE BLOCKS ABOVE THE WASTE
# 12 Block-block 56 - 64
  56 64 0 1 0 2 1 1 0 0
  1
        1
        4.77970E-01
  1
# 13 Block-block 57 - 67
  57 67 0 1 0 2 1 1 0 0
  1
        1
        4.42040E-01
  1
# 14 Block-block 58 - 70
  58 70 0 1 0 2 1 1 0 0
  1
         1
  1
         3.67080E-01
# 15 Block-block 59 - 73
  59 73 0 1 0 2 1 1 0 0
  1
         1
  1
        2.40720E-01
# 16 Block-block 60 - 76
```

```
60 76 0 1 0 2 1 1 0 0
  1 1
  1
       3.65940E-01
# BETWEEN CONCRETE BACKFILL AND CONCRETE BELOW THE WASTE
# 17 Block-block 56 - 79
  56 79 0 1 0 2 1 1 0 0
 1 1
-1 4.10580E-01
# 18 Block-block 57 - 80
  57 80 0 1 0 2 1 1 0 0
 1 1
-1 5.13890E-01
# 19 Block-block 58 - 81
 58 81 0 1 0 2 1 1 0 0
  1 1
 -1 4.11690E-01
# 20 Block-block 59 - 82
  59 82 0 1 0 2 1 1 0 0
  1
         1
  -1 2.03310E-01
# 21 Block-block 60 - 83
 60 83 0 1 0 2 1 1 0 0
  1
          1
  -1 3.32250E-01
# BETWEEN CONCRETE BACKFILL AND FICTITIOUS BLOCK
# 22 Block-block 56 - 84
 56 84 0 1 1 2 1 0 0 0
  1 1
-1 8.17200E-02
# 23 Block-block 57 - 87
 57 87 0 1 1 2 1 0 0 0
  1
-1
          1
        4.92100E-02
# 24 Block-block 58 - 90
 58 90 0 1 1 2 1 0 0 0
  1 1
-1 2 74900E-02
# 25 Block-block 59 - 93
  59 93 0 1 1 2 1 0 0 0
  1 1
-1 2.73400E-02
# 26 Block-block 60 - 96
  60 96 0 1 1 2 1 0 0 0
  1 1
-1 4.74700E-02
# BETWEEN CONCRETE BACKFILL AND FICTITIOUS BLOCK
# 27 Block-block 56 - 99
  56 99 0 1 1 2 1 0 0 0
 1 1
-1 6.15000E-02
# 28 Block-block 57 - 102
 57 102 0 1 1 2 1 0 0 0
 1 1
-1 2.27700E-02
# 29 Block-block 58 - 105
  58 105 0 1 1 2 1 0 0 0
  1 1
-1 6.10800E-02
# 30 Block-block 59 - 108
 59 108 0 1 1 2 1 0 0 0
  1 1
-1 3.97300E-02
# 31 Block-block 60 - 111
 60 111 0 1 1 2 1 0 0 0
```

```
1
          1
  -1 5.01200E-02
# BETWEEN CONCRETE BACKFILL AND FICTITIOUS BLOCK
# 32 Block-block 56 - 114
  56 114 0 1 2 2 0 0 0 1
  1
          1
         5.36000E-03
  -1
 1
 1.1843E-01
# 33 Block-block 57 - 114
  57 114 0 1 2 2 0 0 0 1
  1 1
  1
        5.36000E-03
 1
 1.6607E-01
# BETWEEN CONCRETE BACKFILL AND FICTITIOUS BLOCK
# 34 Block-block 57 - 115
 57 115 0 1 2 2 0 0 0 1
  1 1
 1
        1.38470E-01
 1
 1.6607E-01
# 35 Block-block 58 - 115
  58 115 0 1 2 2 0 0 0 1
  1
          1
  -1
        1.38470E-01
 1
 1.6607E-01
# BETWEEN CONCRETE BACKFILL AND FICTITIOUS BLOCK
# 36 Block-block 58 - 116
  58 116 0 1 2 2 0 0 0 1
  1 1
  1
        2.71650E-01
 1
 1.6607E-01
# 37 Block-block 59 - 116
 59 116 0 1 2 2 0 0 0 1
  1 1
  -1 2.71650E-01
 1
 1.6607E-01
# BETWEEN CONCRETE BACKFILL AND FICTITIOUS BLOCK
# 38 Block-block 59 - 117
  59 117 0 1 2 2 0 0 0 1
  1 1
  1 3.01310E-01
 1
 1.6607E-01
# 39 Block-block 60 - 117
  60 117 0 1 2 2 0 0 0 1
  1 1
  -1 3.01310E-01
 1
 1.6607E-01
#
# BETWEEN CONCRETE BACKFILL AND WALL BETWEEN WASTE AND GRAVEL
BACKFILL (LOADING ZONE)
# 40 Block-block 60 - 123
  60 123 0 1 2 1 1 1 0 0
  1 1
1 3.65220E-01
# WITHIN WALL BETWEEN CONCRETE BACKFILL AND GRAVEL BACKFILL (PASSAGE)
# 41 Block-block 61 - 62
  61 62 0 1 2 2 1 1 0 0
```

```
1
          1
  1
         8.12000E-02
# BETWEN WALL BETWEEN CONCRETE BACKFILL AND GRAVEL BACKFILL (PASSAGE)
AND FICTITIOUS BLOCK
# 42 Block-block 62 - 63
  62 63 0 1 2 2 1 0 0 0
  1
        1
  1 8.12000E-02
# BETWEEN FICTITIOUS BLOCK AND GRAVEL BACKFILL (PASSAGE)
# 43 Block-block 63 - 126
  63 126 0 1 2 2 0 1 0 0
  1 1
 1 8.12000E-02
# WITHIN CONCRETE ABOVE THE WASTE (TRANSVERSE THE MAIN AXIS OF
TUNNEL)
# 44 Block-block 64 - 65
 64 65 0 1 1 1 1 1 0 0
  1 1
  1 4.77970E-01
# 45 Block-block 65 - 66
  65 66 0 1 1 1 1 1 0 0
  1 1
1 4.77970E-01
# 46 Block-block 67 - 68
 67 68 0 1 1 1 1 1 0 0
  1 1
       4.42040E-01
  1
# 47 Block-block 68 - 69
 68 69 0 1 1 1 1 1 0 0
  1 1
  1
       4.42040E-01
# 48 Block-block 70 - 71
  70 71 0 1 1 1 1 1 0 0
  1 1
 1 3.67080E-01
# 49 Block-block 71 - 72
  71 72 0 1 1 1 1 1 0 0
 1 1
1 3.67080E-01
# 50 Block-block 73 - 74
  73 74 0 1 1 1 1 1 0 0
 1 1
1 2.40720E-01
# 51 Block-block 74 - 75
  74 75 0 1 1 1 1 1 0 0
 1 1
1 2.40720E-01
# 52 Block-block 76 - 77
  76 77 0 1 1 1 1 1 0 0
 1 1
1 3.65940E-01
# 53 Block-block 77 - 78
  77 78 0 1 1 1 1 1 0 0
  1 1
1 3.65940E-01
# WITHIN CONCRETE ABOVE THE WASTE (ALONG THE MAIN AXIS OF TUNNEL)
# 54 Block-block 64 - 67
# 55 Block-block 65 - 68
# 56 Block-block 66 - 69
# 57 Block-block 67 - 70
# 58 Block-block 68 - 71
# 59 Block-block 69 - 72
```

```
3.80536E-01
  - 1
# 80 Block-block 190 - 191
 190 191 0 1 1 1 1 1 0 0
  1
          1
 - 1
        3.80536E-01
# WITHIN CONCRETE BELOW THE WASTE (ALONG THE MAIN AXIS OF TUNNEL)
# 81 Block-block 79 - 80
# 82 Block-block 80 - 81
# 83 Block-block 81 - 82
# 84 Block-block 82 - 83
# BETWEEN CONCRETE IN BOTTOM OF THE TUNNEL AND CONCRETE BACKFILL ON
LEFT SIDE OF THE TUNNEL
# 85 Block-block 79 - 149
 79 149 0 1 0 0 1 1 0 0
 1 1
 1 9.36967E-03
# 86 Block-block 79 - 162
  79 162 0 1 0 0 1 1 0 0
  1 1
  1
       9.36967E-03
# 87 Block-block 79 - 163
  79 163 0 1 0 0 1 1 0 0
  1
         1
  1
         9.36967E-03
# 88 Block-block 80 - 150
  80 150 0 1 0 0 1 1 0 0
  1
          1
  1
         8.47567E-03
# 89 Block-block 80 - 164
  80 164 0 1 0 0 1 1 0 0
         1
  1
        8.47567E-03
  1
# 90 Block-block 80 - 165
  80 165 0 1 0 0 1 1 0 0
  1 1
  1
       8.47567E-03
# 91 Block-block 81 - 151
  81 151 0 1 0 0 1 1 0 0
 1 1
1 7.35400E-03
# 92 Block-block 81 - 166
  81 166 0 1 0 0 1 1 0 0
  1 1
1 7.35400E-03
# 93 Block-block 81 - 167
  81 167 0 1 0 0 1 1 0 0
 1 1
1 7.35400E-03
# 94 Block-block 82 - 152
  82 152 0 1 0 0 1 1 0 0
  1 1
1 4.70733E-03
# 95 Block-block 82 - 168
  82 168 0 1 0 0 1 1 0 0
  1 1
1 4.70733E-03
# 96 Block-block 82 - 169
  82 169 0 1 0 0 1 1 0 0
  1 1
1 4.70733E-03
# 97 Block-block 83 - 153
  83 153 0 1 0 0 1 1 0 0
  1 1
1 7.91500E-03
```

```
1
         1
 1 8 18033E-03
# 114 Block-block 83 - 181
  83 181 0 1 0 0 1 1 0 0
  1 1
 1 8.18033E-03
# BETWEEN CONCRETE BELOW THE WASTE AND GRAVEL IN BOTTOM OF THE TUNNEL
# 115 Block-block 183 - 144
 183 144 0 1 1 0 1 1 0 0
  1 1
  -1 4.68130E-01
# 116 Block-block 185 - 145
 185 145 0 1 1 0 1 1 0 0
  1 1
  -1 5.70663E-01
# 117 Block-block 187 - 146
 187 146 0 1 1 0 1 1 0 0
  1
         1
  -1 4.58716E-01
# 118 Block-block 189 - 147
 189 147 0 1 1 0 1 1 0 0
  1
         1
  -1 2.32102E-01
# 119 Block-block 191 - 148
 191 148 0 1 1 0 1 1 0 0
  1
         1
  -1
         3.80536E-01
# WITHIN FICTITIOUS BLOCK (TRANSVERSE TO THE MAIN AXIS OF TUNNEL)
# 120 Block-block 84 - 85
 84 85 0 1 2 2 0 0 0 0
  1
          1
  -1 8.17200E-02
# 121 Block-block 85 - 86
 85 86 0 1 2 2 0 0 0 0
  1 1
-1 8.17200E-02
# 122 Block-block 87 - 88
  87 88 0 1 2 2 0 0 0 0
  1 1
-1 4.92100E-02
# 123 Block-block 88 - 89
  88 89 0 1 2 2 0 0 0 0
  1 1
-1 4.92100E-02
# 124 Block-block 90 - 91
  90 91 0 1 2 2 0 0 0 0
  1 1
  -1
        2.74900E-02
# 125 Block-block 91 - 92
 91 92 0 1 2 2 0 0 0 0
  1 1
  -1
        2.74900E-02
# 126 Block-block 93 - 94
  93 94 0 1 2 2 0 0 0 0
  1 1
-1 2.73400E-02
  -1
# 127 Block-block 94 - 95
  94 95 0 1 2 2 0 0 0 0
  1 1
-1 2.73400E-02
# 128 Block-block 96 - 97
  96 97 0 1 2 2 0 0 0 0
  1 1
-1 4.74700E-02
  1
```

```
# 153 Block-block 108 - 109
  108 109 0 1 2 2 0 0 0 0
  1
          1
  -1 3.97300E-02
# 154 Block-block 109 - 110
  109 110 0 1 2 2 0 0 0 0
  1
          1
  -1 3.97300E-02
# 155 Block-block 111 - 112
  111 112 0 1 2 2 0 0 0 0
  1
          1
  -1 5.01200E-02
# 156 Block-block 112 - 113
  112 113 0 1 2 2 0 0 0 0
  1
          1
  -1
        5.01200E-02
# WITHIN FICTITIOUS BLOCK (ALONG THE MAIN AXIS OF TUNNEL)
# 157 Block-block 99 - 102
# 158 Block-block 100 - 103
# 159 Block-block 101 - 104
# 160 Block-block 102 - 105
# 161 Block-block 103 - 106
# 162 Block-block 104 - 107
# 163 Block-block 105 - 108
# 164 Block-block 106 - 109
# 165 Block-block 107 - 110
# 166 Block-block 108 - 111
# 167 Block-block 109 - 112
# 168 Block-block 110 - 113
# BETWEEN FICTITIOUS BLOCK AND CONCRETE BACKFILL ON RIGHT SIDE OF THE
WASTE
# 169 Block-block 101 - 154
 101 154 0 1 2 1 0 1 0 0
  1 1
  -1 6.15000E-02
# 170 Block-block 104 - 155
  104 155 0 1 2 1 0 1 0 0
  1 1
  -1 2.27700E-02
# 171 Block-block 107 - 156
  107 156 0 1 2 1 0 1 0 0
  1 1
         6.10800E-02
  - 1
# 172 Block-block 110 - 157
  110 157 0 1 2 1 0 1 0 0
  1 1
-1 3.97300E-02
# 173 Block-block 113 - 158
  113 158 0 1 2 1 0 1 0 0
  1 1
-1 5.01200E-02
# WITHIN WALL BETWEEN CONCRETE BACKFILL AND GRAVEL BACKFILL (LOADING
ZONE)
# 174 Block-block 123 - 124
  123 124 0 1 2 2 1 1 0 0
  1 1
1 3.65220E-01
# BETWEN WALL BETWEEN CONCRETE BACKFILL AND GRAVEL BACKFILL (LOADING
ZONE) AND FICTITIOUS BLOCK
# 175 Block-block 124 - 125
 124 125 0 1 2 2 1 0 0 0
  1 1
1 3.65220E-01
# BETWEEN FICTITIOUS BLOCK AND GRAVEL BACKFILL (LOADING ZONE)
```

```
# 176 Block-block 125 - 159
  125 159 0 1 2 2 0 1 0 0
         1
  1
  1
        3.65220E-01
# WITHIN GRAVEL BACKFILL (PASSAGE)
# 177 Block-block 126 - 127
  126 127 0 1 2 2 1 1 0 0
  1
         1
  -1 1.70763E-01
# 178 Block-block 127 - 128
  127 128 0 1 2 2 1 1 0 0
  1
          1
  -1 1.17297E-01
# BETWEEN GRAVEL BACKFILL (PASSAGE) AND GRAVEL IN TOP OF TUNNEL
# 179 Block-block 126 - 129
  126 129 0 1 0 2 1 1 0 0
  1 1
       1.19407E-01
  1
# 180 Block-block 126 - 130
  126 130 0 1 0 2 1 1 0 0
  1
          1
        1.19407E-01
  1
# 181 Block-block 126 - 131
  126 131 0 1 0 2 1 1 0 0
  1
          1
  1
         1.19407E-01
# BETWEEN GRAVEL BACKFILL (PASSAGE) AND GRAVEL IN BOTTOM OF TUNNEL
# 182 Block-block 126 - 144
  126 144 0 1 1 2 1 1 0 0
  1
        1
  -1
         4.54920E-02
# BETWEEN GRAVEL BACKFILL (PASSAGE) AND CONCRETE BACKFILL ON LEFT
SIDE OF STRUCTURE
# 183 Block-block 126 - 149
  126 149 0 1 0 2 1 1 0 0
  1 1
  -1 1.30900E-03
# 184 Block-block 126 - 162
  126 162 0 1 0 2 1 1 0 0
  1 1
  -1 1.30900E-03
# 185 Block-block 126 - 163
  126 163 0 1 0 2 1 1 0 0
  1 1
   -1 1.30900E-03
# BETWEEN GRAVEL BACKFILL (PASSAGE) AND CONCRETE BACKFILL ON RIGHT
SIDE OF STRUCTURE
# 186 Block-block 126 - 154
  126 154 0 1 0 2 1 1 0 0
  1 1
  -1 1.12367E-03
# 187 Block-block 126 - 172
  126 172 0 1 0 2 1 1 0 0
  1 1
  0
        1.12367E-03
# 188 Block-block 126 - 173
  126 173 0 1 0 2 1 1 0 0
  1 1
-1 1.12367E-03
# WITHIN GRAVEL IN TOP OF THE TUNNEL (TRANSVERSE TO THE MAIN AXIS OF
TUNNEL)
# 189 Block-block 129 - 130
 129 130 0 1 0 0 1 1 0 0
  1 1
```

```
# 221 Block-block 138 - 168
 138 168 0 1 1 0 1 1 0 0
 1 1
-1 5.21967E-03
# 222 Block-block 138 - 169
 138 169 0 1 1 0 1 1 0 0
 1 1
 -1 5.21967E-03
# 223 Block-block 141 - 153
 141 153 0 1 1 0 1 1 0 0
 1 1
 -1 8.65800E-03
# 224 Block-block 141 - 170
 141 170 0 1 1 0 1 1 0 0
 1 1
 -1 8.65800E-03
# 225 Block-block 141 - 171
 141 171 0 1 1 0 1 1 0 0
  1 1
        8.65800E-03
 -1
# BETWEEN GRAVEL IN TOP OF THE TUNNEL AND CONCRETE ON RIGHT SIDE OF
THE TUNNEL
# 226 Block-block 129 - 154
 129 154 0 1 1 0 1 1 0 0
 1
         1
  -1
         1.09367E-02
# 227 Block-block 129 - 172
 129 172 0 1 1 0 1 1 0 0
  1
         1
  -1
         1.09367E-02
# 228 Block-block 129 - 173
 129 173 0 1 1 0 1 1 0 0
         1
 1
 -1 1.09367E-02
# 229 Block-block 132 - 155
 132 155 0 1 1 0 1 1 0 0
 1 1
 -1
        9.52433E-03
# 230 Block-block 132 - 174
 132 174 0 1 1 0 1 1 0 0
  1 1
  -1 9.52433E-03
# 231 Block-block 132 - 175
 132 175 0 1 1 0 1 1 0 0
  1 1
  -1
        9.52433E-03
# 232 Block-block 135 - 156
 135 156 0 1 1 0 1 1 0 0
  1 1
 - 1
        3.55533E-02
# 233 Block-block 135 - 176
 135 176 0 1 1 0 1 1 0 0
 1 1
-1 3.55533E-02
# 234 Block-block 135 - 177
 135 177 0 1 1 0 1 1 0 0
 1
-1
         1
         3.55533E-02
# 235 Block-block 138 - 157
 138 157 0 1 1 0 1 1 0 0
 1 1
-1 6.14600E-03
# 236 Block-block 138 - 178
 138 178 0 1 1 0 1 1 0 0
```

```
1
 1
 -1 6.14600E-03
# 237 Block-block 138 - 179
 138 179 0 1 1 0 1 1 0 0
  1 1
  -1 6.14600E-03
# 238 Block-block 141 - 158
 141 158 0 1 1 0 1 1 0 0
         1
  1
  -1 9.01833E-03
# 239 Block-block 141 - 180
 141 180 0 1 1 0 1 1 0 0
  1 1
 -1 9.01833E-03
# 240 Block-block 141 - 181
 141 181 0 1 1 0 1 1 0 0
 1 1
  -1 9.01833E-03
# WITHIN GRAVEL IN BOTTOM OF THE TUNNEL (ALONG THE MAIN AXIS OF
TIINNEL.)
# 241 Block-block 144 - 145
 144 145 0 1 2 2 1 1 0 0
 1 1
  -1 2.55100E-02
# 242 Block-block 145 - 146
 145 146 0 1 2 2 1 1 0 0
  1
         1
  1
        7.32800E-03
# 243 Block-block 146 - 147
 146 147 0 1 2 2 1 1 0 0
  1 1
  1
       1.77070E-02
# 244 Block-block 147 - 148
 147 148 0 1 2 2 1 1 0 0
  1 1
       3.90800E-03
  1
# WITHIN CONCRETE BACKFILL ON LEFT SIDE OF THE TUNNEL (ALONG THE MAIN
AXIS OF TUNNEL)
# 245 Block-block 149 - 150
 149 150 0 1 2 2 1 1 0 0
  1 1
  -1 4.79333E-04
# 246 Block-block 150 - 151
 150 151 0 1 2 2 1 1 0 0
  1 1
 1
       3.09333E-04
# 247 Block-block 151 - 152
 151 152 0 1 2 2 1 1 0 0
  1 1
 1
       2.01000E-04
# 248 Block-block 152 - 153
 152 153 0 1 2 2 1 1 0 0
  1 1
 1 6.76667E-05
# 249 Block-block 162 - 164
 162 164 0 1 2 2 1 1 0 0
  1 1
-1 4.79333E-04
# 250 Block-block 163 - 165
 163 165 0 1 2 2 1 1 0 0
  1 1
-1 4.79333E-04
# 251 Block-block 164 - 166
 164 166 0 1 2 2 1 1 0 0
```

```
1 1
1 3.09333E-04
# 252 Block-block 165 - 167
 165 167 0 1 2 2 1 1 0 0
 1 1
1 3.09333E-04
# 253 Block-block 166 - 168
 166 168 0 1 2 2 1 1 0 0
  1
       1
 1 2.01000E-04
# 254 Block-block 167 - 169
 167 169 0 1 2 2 1 1 0 0
  1 1
  1
       2.01000E-04
# 255 Block-block 168 - 170
 168 170 0 1 2 2 1 1 0 0
 1 1
       6.76667E-05
  1
# 256 Block-block 169 - 171
 169 171 0 1 2 2 1 1 0 0
  1 1
      6.76667E-05
 1
# WITHIN CONCRETE BACKFILL ON LEFT SIDE OF THE TUNNEL (TRANSVERSE TO
THE MAIN AXIS OF TUNNEL)
# 257 Block-block 149 - 162
 149 162 0 1 1 1 1 1 0 0
  1
         1
  -1
         8.50420E-02
# 258 Block-block 162 - 163
 162 163 0 1 1 1 1 1 0 0
  1
         1
         8.83630E-02
  - 1
# 259 Block-block 150 - 164
 150 164 0 1 1 1 1 1 0 0
  1
         1
  - 1
        7.65003E-02
# 260 Block-block 164 - 165
 164 165 0 1 1 1 1 1 0 0
  1 1
  -1 1.03787E-01
# 261 Block-block 151 - 166
  151 166 0 1 1 1 1 1 0 0
  1 1
-1 2.76357E-02
# 262 Block-block 166 - 167
 166 167 0 1 1 1 1 1 0 0
  1 1
  - 1
        2.77833E-02
# 263 Block-block 152 - 168
  152 168 0 1 1 1 1 1 0 0
  1 1
  -1
         2.77160E-02
# 264 Block-block 168 - 169
 168 169 0 1 1 1 1 1 0 0
  1 1
        2.80950E-02
  -1
# 265 Block-block 153 - 170
 153 170 0 1 1 1 1 1 0 0
  1
-1
         1
         4.90357E-02
# 266 Block-block 170 - 171
 170 171 0 1 1 1 1 1 0 0
  1 1
-1 5.05983E-02
```

```
2.20433E-02
  - 1
# 282 Block-block 174 - 175
 174 175 0 1 1 1 1 1 0 0
  1
         1
  -1 2.13177E-02
# 283 Block-block 156 - 176
 156 176 0 1 1 1 1 1 0 0
  1
         1
  -1 8.89447E-02
# 284 Block-block 176 - 177
 176 177 0 1 1 1 1 1 0 0
  1
         1
  -1 1.16811E-01
# 285 Block-block 157 - 178
 157 178 0 1 1 1 1 1 0 0
  1
         1
  - 1
        4.06510E-02
# 286 Block-block 178 - 179
 178 179 0 1 1 1 1 1 0 0
  1
          1
  -1 4.15690E-02
# 287 Block-block 158 - 180
 158 180 0 1 1 1 1 1 0 0
  1
          1
  -1
         5.16787E-02
# 288 Block-block 180 - 181
 180 181 0 1 1 1 1 1 0 0
  1
          1
  -1
        5.32353E-02
# WITHIN GRAVEL IN LOADING ZONE
# 289 Block-block 159 - 160
 159 160 0 1 2 2 1 1 0 0
  1
          1
  -1 7.25100E-02
# 290 Block-block 160 - 161
  160 161 0 1 2 2 1 1 0 0
  1
          1
  -1
        4.25800E-02
# BETWEEN GRAVEL IN LOADING ZONE AND GRAVEL IN TOP OF TUNNEL
# 291 Block-block 159 - 141
  159 141 0 1 0 2 1 1 0 0
  1 1
  1 1.69803E-01
# 292 Block-block 159 - 142
  159 142 0 1 0 2 1 1 0 0
  1 1
1 1.69803E-01
  1
# 293 Block-block 159 - 143
  159 143 0 1 0 2 1 1 0 0
  1 1
1 1.69803E-01
# BETWEEN GRAVEL IN LOADING ZONE AND GRAVEL IN BOTTOM OF TUNNEL
# 294 Block-block 159 - 148
 159 148 0 1 0 2 1 1 0 0
  1 1
-1 3.65700E-02
# BETWEEN GRAVEL IN LOADING ZONE AND CONCRETE BACKFILL ON LEFT SIDE
OF TUNNEL
# 295 Block-block 159 - 153
  159 153 0 1 0 2 1 1 0 0
  1
-1
          1
         8.87333E-04
# 296 Block-block 159 - 170
  159 170 0 1 0 2 1 1 0 0
```

```
1
          1
  -1 8.87333E-04
# 297 Block-block 159 - 171
  159 171 0 1 0 2 1 1 0 0
  1
          1
  -1
         8.87333E-04
# BETWEEN GRAVEL IN LOADING ZONE AND CONCRETE BACKFILL ON RIGHT SIDE
OF TUNNEL
# 298 Block-block 159 - 158
  159 158 0 1 0 2 1 1 0 0
  1
          1
  -1 8.40000E-04
# 299 Block-block 159 - 180
  159 180 0 1 0 2 1 1 0 0
  1
         1
  -1 8.40000E-04
# 300 Block-block 159 - 181
  159 181 0 1 0 2 1 1 0 0
  1
          1
  -1 8.40000E-04
# BETWEEN CONCRETE BACKFILL AND CEMENT BETWEEN DRUM IN DRUM WITH ASHES
# 301 Block-block 56 - 192
  56 192 0 1 0 1 1 1 0 0
  1 1
  -1
         5.59165E-01
# 302 Block-block 56 - 193
  56 193 0 1 0 1 1 1 0 0
  1 1
  1
        5.59165E-01
# BETWEEN CONCRETE BACKFILL AND CONCRETE CONTAINER WITHOUT ACTIVITY
(ACTIVITY MOVED TO BMA)
# 303 Block-block 56 - 194
  56 194 0 1 2 2 1 1 0 0
  1 1
        5.59165E-01
  1
# 304 Block-block 56 - 195
  56 195 0 1 2 2 1 1 0 0
  1 1
  -1 5.59165E-01
# WITHIN CONCRETE CONTAINER WITHOUT ACTIVITY (ACTIVITY MOVED TO BMA)
# 305 Block-block 194 - 195
  194 195 0 1 2 2 0 0 0 0
          1
  1
  1
         5.59165E-01
```

	/ERSION INPU	JT DATA F	ORMAT			
ITYPE 1	NGROUP 1	NSINKS 0	IWFLOW 1	NSOURC 50	NBSIZE 0	ISPS 0
Flow st	ceam condit:	lons				
NINFLW 36	NUTFLW 1.3		% if IWFL	.OW = 1		
Inflow o	conditions	(one line	per inflow	7) 0		
QINFW QINFW	,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,,	NBWIN	NBNWIN	% 1Í IW	FLOW = 1	
0.03124	************	120	1			
0.07188	33333333333333	128	1			
0.07635		129	1			
0.07635		130	1			
0.07635		131	1			
0.11984	666666667	132	1			
0.11984	6666666667	133	1			
0.11984		134	1			
0.56966	6666666667	136	1			
0.56966	6666666667	137	1			
0.03429	5333333333333	138	1			
0.03429	5333333333333	139	1			
0.03429	53333333333333	140	1			
0.02468	3	141	1			
0.02468	3	142	1			
0.02468	3	143	1			
0.000000		145	1			
0.73212	3	146	1			
0.21793		147	1			
0.26795	2	148	1			
0.18256	33333333333	159	1			
0.18256	33333333333	160	1			
0.20736.	33333333333	161	1			
0.112/3	£	165	1			
0 05195	>	167	1			
0.032214	1	169	1			
0.03618	7	171	1			
0.09783	Э	173	1			
0.03314	7	175	1			
0.14214	9	177	1			
0.043713	3	179	1			
0.04346.	5	181	T			
Outflow	conditions	(one lin	e per inflo	w)		
QUTFW		NBWOUT	NBNWOUT	% if IW	FLOW = 1	
0.00503	33333333333333	3 126	1			
0.00503	33333333333333	3 127	1			
0.00503	333333333333333	3 128	1			
0.30393		131	1			
0.02274		125	1			
U.134/50	1 00000000/	133	1			

5.06469666666667 137 1 0.068019 140 1 0.143864 143 1 0.07633 159 1 0.07633 160 1 0.07633 161 1 # Number of blocks connections and materials NBLOCK NBCOUP NMAT # 191 270 5 # Data on materials (one line per material) # Material 1 = Waste # Material 2 = Construction concrete Material 3 = Water # # Material 4 = Gravel Material 5 = Porous concrete # MAT DENSM PORM # 2250.0 0.633 1 2 2529.4 0.150 1000.0 1.000 3 2700.0 0.300 4 5 2625.0 0.200 # Data on time series TINIT AGEO RAT TEND NTERM LREAD # 0.50 1.23 1000.00 90 0.00 0 -# Data on numerical parameters # EPS MSPAR EWT NLOOP # 1.00E-06 1.00E-18 3 1 # Data on group (one set of data per group common for all sources) # # Group No: 1 # NNCH ISPEC ISPSOL 1 0 1 # # Nuclide: C-14 org # TAU CSOLUB AINV 5.7300E+03 1.00E+10 1.2784E-02 # % if ISPEC = 1 # CIRF # # # MAT DEFM SKDM % one line per material 0.06307 1 0.000 0.00032 0.000 2 3 0.06307 0.000 0.000 4 0.01892 0.00315 0.000 5 # # Data on sources # NSBCHA ISEQ MAT IFWATB VOLSOU FINVEN JINVEN # # IBLOCK ISSEO % if ISEQ = 1 # % if JINVEN = 1. One line for each group. # XXXX # IBSOUR % if ISEQ = 0 #

```
#
# IWINT
                     % if IFWATB = 1
#
# WBFLOW
                      % if IFWATB = 1 and IWINT = 0
#
# Waste
# Source 1
 1 0 1 0 0.0001 1.0000000E+00 1
 0.000000E+00
 1
# Source 2
 1 0 1 1 2064.0000 1.0000000E+00 1
#
 4.300000E-01
 2
 0
8.183800E-01
# Source 3
1 0 1 0 0.0001 1.000000E+00 1
#
0.000000E+00
3
# Source 4
 1 0 1 0 0.0001 1.0000000E+00 1
#
 0.000000E+00
 4
# Source 5
 1 0 1 0 0.0001 1.000000E+00 1
#
 0.000000E+00
 5
# Source 6
 1 0 1 0 0.0001 1.0000000E+00 1
#
 0.000000E+00
 6
# Source 7
 1 0 1 0 0.0001 1.0000000E+00 1
#
 0.000000E+00
 7
# Source 8
 1 0 1 0 0.0001 1.0000000E+00 1
#
 0.000000E+00
 8
# Source 9
 1 0 1 0 0.0001 1.000000E+00 1
#
 0.000000E+00
 9
# Source 10
1 0 1 0 0.0001 1.000000E+00 1
#
0.000000E+00
 10
# Source 11
 1 0 1 0 0.0001 1.000000E+00 1
 0.000000E+00
 11
# Source 12
```

```
1 0 1 1 576.0000 1.0000000E+00 1
#
 1.200000E-01
 12
 0
4.089400E-01
# Source 13
 1 0 1 0 0.0001 1.0000000E+00 1
0.000000E+00
13
# Source 14
 1 0 1 0 0.0001 1.0000000E+00 1
#
0.000000E+00
14
# Source 15
1 0 1 0 0.0001 1.000000E+00 1
#
0.000000E+00
 15
# Source 16
 1 0 1 0 0.0001 1.0000000E+00 1
#
0.000000E+00
 16
# Source 17
 1 0 1 0 0.0001 1.0000000E+00 1
#
0.000000E+00
 17
# Source 18
1 0 1 0 0.0001 1.000000E+00 1
#
0.000000E+00
18
# Source 19
 1 0 1 0 0.0001 1.0000000E+00 1
#
0.000000E+00
19
# Source 20
 1 0 1 0 0.0001 1.000000E+00 1
#
0.000000E+00
20
# Source 21
1 0 1 0 0.0001 1.0000000E+00 1
#
0.000000E+00
 21
# Source 22
 1 0 1 1 576.0000 1.0000000E+00 1
#
 1.200000E-01
 22
 0
6.729000E-01
# Source 23
 1 0 1 0 0.0001 1.0000000E+00 1
#
0.000000E+00
 23
# Source 24
```

```
1 0 1 0 0.0001 1.0000000E+00 1
#
 0.000000E+00
 24
# Source 25
 1 0 1 0 0.0001 1.0000000E+00 1
0.000000E+00
25
# Source 26
1 0 1 0 0.0001 1.0000000E+00 1
#
0.000000E+00
26
# Source 27
 1 0 1 0 0.0001 1.0000000E+00 1
#
0.000000E+00
27
# Source 28
1 0 1 0 0.0001 1.0000000E+00 1
#
0.000000E+00
 2.8
# Source 29
 1 0 1 0 0.0001 1.0000000E+00 1
#
 0.000000E+00
 29
# Source 30
 1 0 1 0 0.0001 1.0000000E+00 1
#
0.000000E+00
 3.0
# Source 31
 1 0 1 0 0.0001 1.000000E+00 1
#
 0.000000E+00
 31
# Source 32
 1 0 1 1 624.0000 1.000000E+00 1
 1.3000000E-01
 32
 0
 5.264500E-01
# Source 33
 1 0 1 0 0.0001 1.0000000E+00 1
#
 0.000000E+00
 33
# Source 34
1 0 1 0 0.0001 1.000000E+00 1
#
0.000000E+00
 34
# Source 35
 1 0 1 0 0.0001 1.0000000E+00 1
 0.000000E+00
 35
# Source 36
1 0 1 0 0.0001 1.0000000E+00 1
#
```

0.000000E+00 36 # Source 37 1 0 1 0 0.0001 1.000000E+00 1 # 0.000000E+00 37 # Source 38 1 0 1 0 0.0001 1.0000000E+00 1 # 0.000000E+00 38 # Source 39 1 0 1 0 0.0001 1.0000000E+00 1 # 0.000000E+00 39 # Source 40 1 0 1 0 0.0001 1.000000E+00 1 # 0.000000E+00 40 # Source 41 1 0 1 0 0.0001 1.0000000E+00 1 # 0.000000E+00 41 # Source 42 1 0 1 1 960.0000 1.000000E+00 1 # 2.000000E-01 42 0 5.437200E-01 # Source 43 1 0 1 0 0.0001 1.000000E+00 1 # 0.000000E+00 43 # Source 44 1 0 1 0 0.0001 1.000000E+00 1 # 0.000000E+00 44 # Source 45 1 0 1 0 0.0001 1.0000000E+00 1 # 0.000000E+00 45 # Source 46 1 0 1 0 0.0001 1.0000000E+00 1 # 0.000000E+00 46 # Source 47 1 0 1 0 0.0001 1.0000000E+00 1 # 0.000000E+00 47 # Source 48 1 0 1 0 0.0001 1.0000000E+00 1 0.000000E+00 48

```
# Source 49
 1 0 1 0 0.0001 1.000000E+00 1
0 000000E+00
49
# Source 50
1 0 1 0 0.0001 1.0000000E+00 1
 0.000000E+00
 50
# Description of blocks not considered before
# Block i
# NZ NY NX NBLSP MAT IQFLOW
# ZADIM YADIM XADIM VOLBLK % one line per compartment if NBLSP = 1
# IWINT
                      % if IOFLOW = 1
# NODWIN NODWUT BFLOW % if IQFLOW = 1 and IWINT = 0
# Walls of concrete tanks 51-55 and 118-122
# Block 51
 1 1 1 0 2 1
 1.0000E+05 3.9641E-05 2.9324E-05 6.8800E+02
 0
 1
          1 0.81838
# Block 52
 1 1 1 0 2 1
 1.0000E+05 1.4205E-04 1.0508E-04
                                 1.9200E+02
 0
          1
                 0.40894
 1
# Block 53
 1 1 1 0 2 1
 1.0000E+05 1.4205E-04 1.0508E-04
                                 1.9200E+02
 0
1
          1
                   0.6729
# Block 54
1 1 1 0 2 1
 1.0000E+05 1.3112E-04 9.6994E-05
                                 2.0800E+02
 0
       1 0.52645
1
# Block 55
1 1 1 0 2 1
1.0000E+05 8.5227E-05 6.3046E-05
                                 3.2000E+02
 0
      1
                 0.54372
1
# Porous concrete
# Block 56
1 1 1 0 5 1
8.6897E-05 2.4157E-04 1.4215E-05
                                 7.4390E+02
1
# Block 57
1 1 1 0 5 1
3.1138E-04 8.6564E-04
                     5.0937E-05
                                  2.0760E+02
1
# Block 58
1 1 1 0 5 1
 3.1138E-04 8.6564E-04
                     5.0937E-05
                                   2.0760E+02
 1
# Block 59
1 1 1 0 5 1
 2.8743E-04 7.9905E-04
                     4.7019E-05
                                 2.2490E+02
 1
```

# Block 60 1 1 1 0 5 1 1.8683E-04 5.1938E-04 3.0562E-05 3.4600E+02 1 # Short side of structure (passage) # Block 61 1 1 1 0 2 1 1.0000E+05 3.5971E-03 3.1010E-03 2.0155E+01 0 1 1 0.17863 # Block 62 1 1 1 0 2 1 1.0000E+05 3.1010E-03 3.1010E-03 2.0155E+01 0 1 1 0.17863 # Fictitious block 63 # Block 63 1 1 1 0 2 1 1.0000E-05 1.0000E-05 1.0000E-05 1.0000E-02 0 1 1 0.17863 # Ceiling # Block 64 1 1 1 0 2 1 8.6586E-01 3.1606E-04 3.1868E-04 2.2314E+02 0 1 1 0.63975 # Block 65 1 1 1 0 2 1 8.6586E-01 3.1606E-04 1.0000E+05 2.2314E+02 0 1 1 0.63975 # Block 66 1 1 1 0 2 1 8.6586E-01 3.1606E-04 1.0000E+05 2.2314E+02 0 1 1 0.63975 # Block 67 1 1 1 0 2 1 3.1027E+00 1.1419E-03 1.1419E-03 6.2272E+01 0 1 1 0.31853 # Block 68 1 1 1 0 2 1 3.1027E+00 1.1419E-03 1.0000E+05 6.2272E+01 0 0.31853 1 1 # Block 69 1 1 1 0 2 1 3.1027E+00 1.1419E-03 1.0000E+05 6.2272E+01 0 1 0.31853 1 # Block 70 1 1 1 0 2 1 3.1027E+00 1.1419E-03 1.1419E-03 6.2272E+01 0 1 1 0.4312 # Block 71 1 1 1 0 2 1 3.1027E+00 1.1419E-03 1.0000E+05 6.2272E+01 0 1 1 0.4312 # Block 72 1 1 1 0 2 1

```
3.1027E+00 1.1419E-03 1.0000E+05 6.2272E+01
 0
1
         1 0.4312
# Block 73
1 1 1 0 2 1
 2.8640E+00 1.0541E-03 1.0541E-03 6.7461E+01
 0
1 1 0.25631
# Block 74
1 1 1 0 2 1
 2.8640E+00 1.0541E-03 1.0000E+05
                               6.7461E+01
 Ο
1
         1 0.25631
# Block 75
1 1 1 0 2 1
2.8640E+00 1.0541E-03 1.0000E+05
                              6.7461E+01
0
1
         1 0.25631
# Block 76
1 1 1 0 2 1
1.8616E+00 6.7315E-04 6.8517E-04
                               1.0379E+02
0
         1 0.22832
1
# Block 77
 1 1 1 0 2 1
 1.8616E+00 6.7315E-04 1.0000E+05
                                1.0379E+02
 0
         1 0.22832
1
# Block 78
 1 1 1 0 2 1
 1.8616E+00 6.7315E-04 1.0000E+05
                               1.0379E+02
 0
         1 0.22832
1
# Bottom of structure 79-83 and 182-191
# Block 79
1 1 1 0 2 1
 8.2372E-03 1.4943E-04 1.5934E-04
                              1 1897E+02
1
# Block 80
1 1 1 0 2 1
 2.9762E-02 5.3990E-04 5.7097E-04
                               3.2928E+01
1
# Block 81
1 1 1 0 2 1
 2.9762E-02 5.3990E-04
                    5.7097E-04
                               3.2928E+01
1
# Block 82
1 1 1 0 2 1
2.7473E-02 4.9837E-04
                    5.2705E-04
                               3.5672E+01
1
# Block 83
1 1 1 0 2 1
1.7544E-02 3.1826E-04
                   3.4258E-04
                              5.5860E+01
1
# Fiktiva block 84-113
# Block 84
1 1 1 0 3 1
1.0000E-05 1.0000E+05 1.0000E-05
                               1.0000E-02
 0
1 1 0.10248
# Block 85
1 1 1 0 3 1
1.0000E-05 1.0000E+05 1.0000E-05
                              1.0000E-02
 0
```

1 1 0.10248 # Block 86 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.10248 # Block 87 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.11733 # Block 88 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1 0000E-02 0 1 0.11733 1 # Block 89 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1 0000E-02 0 1 1 0.11733 # Block 90 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.0106 # Block 91 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.0106 # Block 92 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.0106 # Block 93 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.03144 # Block 94 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 0.03144 1 # Block 95 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 0.03144 # Block 96 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.03189 # Block 97 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 0 1 1 0.03189 # Block 98 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02

0 1 1 0.03189 # Fictitious blocks (Wall on right side of structure) 99-113 # Block 99 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Block 100 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Block 101 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Block 102 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Block 103 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Block 104 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 # Block 105 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 # Block 106 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Block 107 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Block 108 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Block 109 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Block 110 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Block 111 1 1 1 0 3 1 1.0000E-05 1.0000E-05 1.0000E+05 1.0000E-02 1 # Block 112 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Block 113 1 1 1 0 3 1 1.0000E-05 1.0000E+05 1.0000E-05 1.0000E-02 1 # Fictitious block 114-117

# Block 114 1 1 1 0 3 1 1.0000E+05 1.0000E+05 1.0000E+05 1.0000E-02 0 1 1 0.07433 # Block 115 1 1 1 0 3 1 1.0000E+05 1.0000E+05 1.0000E+05 1.0000E-02 0 1 1 0.01608 # Block 116 1 1 1 0 3 1 1.0000E+05 1.0000E+05 1.0000E+05 1.0000E-02 0 0.2417 1 1 # Block 117 1 1 1 0 3 1 1.0000E+05 1.0000E+05 1.0000E+05 1.0000E-02 0 0.27014 1 1 # Walls of conrete tanks 118-122 # Block 118 1 1 1 0 2 1 1.0000E+05 3.9641E-05 2.9324E-05 6.8800E+02 0 1 1 0.81838 # Block 119 1 1 1 0 2 1 1.0000E+05 1.4205E-04 1.0508E-04 1.9200E+02 0 1 1 0.40894 # Block 120 1 1 1 0 2 1 1.0000E+05 1.4205E-04 1.0508E-04 1.9200E+02 0 1 0.6729 1 # Block 121 1 1 1 0 2 1 1.0000E+05 1.3112E-04 9.6994E-05 2.0800E+02 0 1 1 0.52645 # Block 122 1 1 1 0 2 1 1.0000E+05 8.5227E-05 6.3046E-05 3.2000E+02 0 1 1 0.54372 # Short side of structure (loading zone) # Block 123 1 1 1 0 2 1 1.0000E+05 3.5971E-03 3.1010E-03 2.0155E+01 0 0.3154 1 1 # Block 124 1 1 1 0 2 1 1.0000E+05 3.1010E-03 3.1010E-03 2.0155E+01 0 0.3154 1 1 # Fictitious block 125 # Block 125 1 1 1 0 2 1 1.0000E+05 1.0000E+05 1.0000E+05 1.0000E-02 0 1 1 0.3154 # Gravel at the end of the tunnel (passage) 126-128

# Block 126 1 1 1 0 4 1 1.4506E-01 1.6538E-02 1.0829E-02 1 6417E+02 1 # Block 127 1 1 1 0 4 1 1.0000E+05 1.0000E+05 1.0829E-02 1.6417E+02 1 # Block 128 1 1 1 0 4 1 1.0000E+05 1.0000E+05 1.0829E-02 1.6417E+02 1 # Gravel on top of the structure 129-143 # Block 129 1 1 1 0 4 1 7.0076E-04 7.7259E-02 6.6038E+00 5 5793E+02 1 # Block 130 1 1 1 0 4 1 6.6038E+00 7.0076E-04 1.0000E+05 5.5793E+02 1 # Block 131 1 1 1 0 4 1 7.0076E-04 1.0000E+05 6.6038E+00 5.5793E+02 # Block 132 1 1 1 0 4 1 2.5319E-03 2.7914E-01 1.8277E+00 1.5442E+02 # Block 133 1 1 1 0 4 1 2.5319E-03 1.0000E+05 1.8277E+00 1.5442E+02 # Block 134 1 1 1 0 4 1 2.5319E-03 1.0000E+05 1.8277E+00 1.5442E+02 1 # Block 135 1 1 1 0 4 1 2.5319E-03 2.7914E-01 1.8277E+00 1.5442E+02 1 # Block 136 1 1 1 0 4 1 2.5319E-03 1.0000E+05 1.8277E+00 1.5442E+02 1 # Block 137 1 1 1 0 4 1 2.5319E-03 1.0000E+05 1.8277E+00 1.5442E+02 1 # Block 138 1 1 1 0 4 1 2.3372E-03 2.5767E-01 1.9801E+00 1.6729E+02 1 # Block 139 1 1 1 0 4 1 2.3372E-03 1.0000E+05 1.9801E+00 1.6729E+02 1 # Block 140 1 1 1 0 4 1 2.3372E-03 1.0000E+05 1.9801E+00 1.6729E+02 1 # Block 141 1 1 1 0 4 1 1.4925E-03 1.6455E-01 3.1006E+00 2.6196E+02

1 # Block 142 1 1 1 0 4 1 1.4925E-03 1.0000E+05 3.1006E+00 2.6196E+02 1 # Block 143 1 1 1 0 4 1 1.4925E-03 1.0000E+05 3.1006E+00 2.6196E+02 1 # Gravel in bottom of tunnel 144-148 # Block 144 1 1 1 0 4 1 3.3621E-04 1.0000E+05 1.3651E+01 2.6769E+02 1 # Block 145 1 1 1 0 4 1 1.2148E-03 1.0000E+05 3.8095E+00 7.4088E+01 1 # Block 146 1 1 1 0 4 1 1.2148E-03 1.0000E+05 3.8095E+00 7.4088E+01 1 # Block 147 1 1 1 0 4 1 1.1213E-03 1.0000E+05 4.1270E+00 8.0262E+01 1 # Block 148 1 1 1 0 4 1 7.1608E-04 1.0000E+05 6.3492E+00 1.2569E+02 1 # Concrete on left side 149-153 and 162-171 # Block 149 1 1 1 0 5 1 7.1664E-01 3.7872E-04 7.7845E+01 4.6941E+01 1 # Block 150 1 1 1 0 5 1 2.5893E+00 1.3684E-03 2.1724E+01 1.2992E+01 1 # Block 151 1 1 1 0 5 1 2.5893E+00 1.3684E-03 2.1724E+01 1.2992E+01 1 # Block 152 1 1 1 0 5 1 2.3901E+00 1.2631E-03 2.3534E+01 1.4075E+01 1 # Block 153 1 1 1 0 5 1 1.5263E+00 8.0661E-04 3.6207E+01 2.2040E+01 1 # Concrete on right side 154-158 and 172-181 # Block 154 1 1 1 0 5 1 7.1664E-01 3.7872E-04 7.7845E+01 4.6941E+01 1 # Block 155 1 1 1 0 5 1 2.5893E+00 1.3684E-03 2.1724E+01 1.2992E+01 1 # Block 156 1 1 1 0 5 1 2.5893E+00 1.3684E-03 2.1724E+01 1.2992E+01 1

# Block 157 1 1 1 0 5 1 2.3901E+00 1.2631E-03 2.3534E+01 1 4075E+011 # Block 158 1 1 1 0 5 1 1.5263E+00 8.0661E-04 3.6207E+01 2.2040E+01 1 # Gravel at the end of the tunnel (loading zone) 159-161 # Block 159 1 1 1 0 4 1 5.4397E-01 6.2019E-02 4.0609E-02 6.1563E+02 1 # Block 160 1 1 1 0 4 1 1.0000E+05 1.0000E+05 4.0609E-02 6.1563E+02 1 # Block 161 1 1 1 0 4 1 1.0000E+05 1.0000E+05 4.0609E-02 6.1563E+02 # Concrete on left side 149-153 and 162-171 # Block 162 1 1 1 0 5 1 1.0000E+05 3.7872E-04 7.7845E+01 4.6941E+01 # Block 163 1 1 1 0 5 1 1.0000E+05 3.7872E-04 7.7845E+01 4.6941E+01 # Block 164 1 1 1 0 5 1 1.0000E+05 1.3684E-03 1.2992E+01 2.1724E+01 1 # Block 165 1 1 1 0 5 1 1.0000E+05 1.3684E-03 2.1724E+01 1 2992E+01 1 # Block 166 1 1 1 0 5 1 1.0000E+05 1.3684E-03 2.1724E+01 1.2992E+01 1 # Block 167 1 1 1 0 5 1 1.0000E+05 1.3684E-03 2.1724E+01 1.2992E+01 1 # Block 168 1 1 1 0 5 1 1.0000E+05 1.2631E-03 2.3534E+01 1.4075E+01 1 # Block 169 1 1 1 0 5 1 1.4075E+01 1.0000E+05 1.2631E-03 2.3534E+01 1 # Block 170 1 1 1 0 5 1 1.0000E+05 8.0661E-04 3.6207E+01 2.2040E+01 1 # Block 171 1 1 1 0 5 1 1.0000E+05 8.0661E-04 3.6207E+01 2.2040E+01 # Concrete on right side 154-158 and 172-181 # Block 172

1 1 1 0 5 1 1.0000E+05 3.7872E-04 7.7845E+01 4.6941E+01 1 # Block 173 1 1 1 0 5 1 1.0000E+05 3.7872E-04 7.7845E+01 4.6941E+01 1 # Block 174 1 1 1 0 5 1 1.0000E+05 1.3684E-03 2.1724E+01 1.2992E+01 1 # Block 175 1 1 1 0 5 1 1.0000E+05 1.3684E-03 2.1724E+01 1.2992E+01 1 # Block 176 1.0000E+05 1.3684E-03 2.1724E+01 1.2992E+01 1 # Block 177 1 1 1 0 5 1 1.0000E+05 1.3684E-03 2.1724E+01 1.2992E+01 1 # Block 178 1 1 1 0 5 1 1.0000E+05 1.2631E-03 2.3534E+01 1.4075E+01 1 # Block 179 1 1 1 0 5 1 1.0000E+05 1.2631E-03 2.3534E+01 1.4075E+01 1 # Block 180 1 1 1 0 5 1 1.0000E+05 8.0661E-04 3.6207E+01 2.2040E+01 1 # Block 181 1 1 1 0 5 1 1.0000E+05 8.0661E-04 3.6207E+01 2.2040E+01 1 # Bottom of structure 79-83 and 182-191 # Block 182 1 1 1 0 2 1 8.2372E-03 1.4943E-04 3.0969E+01 1.1897E+02 0 1 1 0.631108 # Block 183 1 1 1 0 2 1 8.2372E-03 1.4943E-04 3.0969E+01 1.1897E+02 0 1 1 0.631108 # Block 184 1 1 1 0 2 1 2.9762E-02 5.3990E-04 8.5714E+00 3.2928E+01 0 1 0.293864 1 # Block 185 1 1 1 0 2 1 2.9762E-02 5.3990E-04 8.5714E+00 3.2928E+01 0 1 1 0.293864 # Block 186 1 1 1 0 2 1 2.9762E-02 5.3990E-04 8.5714E+00 3.2928E+01 0

```
1
        1
                      0 6763
# Block 187
 1 1 1 0 2 1
 2.9762E-02 5.3990E-04 8.5714E+00
                                     3 2928E+01
 0
 1
                        0.6763
           1
# Block 188
 1 1 1 0 2 1
 2.7473E-02 4.9837E-04
                        9.2857E+00
                                       3.5672E+01
 0
 1
           1
                        0.242692
# Block 189
 1 1 1 0 2 1
 2.7473E-02 4.9837E-04
                        9.2857E+00
                                      3 5672E+01
 0
 1
           1
                        0.242692
# Block 190
 1 1 1 0 2 1
                        1.4541E+01
 1.7544E-02 3.1826E-04
                                       5 5860E+01
 0
 1
                        0.232857
             1
# Block 191
 1 1 1 0 2 1
 1.7544E-02 3.2394E-04
                        1.4541E+01
                                      5.5860E+01
 0
 1
           1
                        0.232857
# Codifying the connections
#
# NSSCO NSBCO NBBCO
  0
         10
                 260
# Connection source - block
#
# IBSOU IBSB ISEQC NABCCO IRZB ICRB IPLUG IRADD
# ICOMPB ICSEQB % if NABCCO > 1 and ISEQC =1
# IDCB
               % if ISEOC = 0. One value for each connection.
# LSCFW OWATIN % if IFWATB(source term) = 1 and IOFLOW(block) = 1
# ISERNU
               % if IRADD =1
# XRADD
               % if IRADD = 1 and ISERNU = 1
# XRADD
               % if IRADD = 1 and ISERNU = 0. One line per group.
#
# BETWEEN WASTE - CONCRETE WALLS OF CONCRETE CONTAINERS
# 1 Source-block 2 - 51
   2 51 0 1 1 1 0 0
 1
 1 0.81838
# 2 Source-block 2 - 118
   2 118 0 1 1 1 0 0
 1
 -1 0.81838
# 3 Source-block 12 - 52
   12 52 0 1 1 1 0 0
 1
 1 0.40894
# 4 Source-block 12 - 119
   12 119 0 1 1 1 0 0
 1
```

```
-1 0.40894
# 5 Source-block 22 - 53
   22 53 0 1 1 1 0 0
 1
 1 0.6729
# 6 Source-block 22 - 120
   22 120 0 1 1 1 0 0
 1
 -1 0.6729
# 7 Source-block 32 - 54
   32 54 0 1 1 1 0 0
 1
 1 0.52645
# 8 Source-block 32 - 121
   32 121 0 1 1 1 0 0
 1
 -1 0.52645
# 9 Source-block 42 - 55
   42 55 0 1 1 1 0 0
 1
 1 0.54372
# 10 Source-block 42 - 122
   42 122 0 1 1 1 0 0
 1
 -1 0.54372
# Connections between blocks not considered as sources
# IBA IBB ISEQC NABCCO IRZA IRZB ICRA ICRB IPLUG IRADD
                               % if NABCCO = 1
# IDCA
         IDCB
 ICOMPA ICOMPB ICSEQA ICSEQB
                               % if NABCCO > 1 and ISQC = 1
#
                                % if NABCCO > 1 and ISOC = 0
#
 TDCA
# TDCB
                                % if NABCCO > 1 and ISOC = 0
# LSCFW
          OWATIN
                           % if IOFLOW(A) = 1 and IOFLOW(B) = 1
# ISERNU
                            % if IRADD = 1
                 % if IRADD = 1 and ISERNU = 0. One line per group
 XRADD
# BETWEEN WALLS OF CONCRETE CONTAINERS AND BACKFILL CONCRETE
# 1 Block-block 51 - 56
  51 56 0 1 2 2 1 1 0 0
   1
          1
   1
         8.18380E-01
# 2 Block-block 52 - 57
  52 57 0 1 2 2 1 1 0 0
   1
           1
         4.08940E-01
  1
# 3 Block-block 53 - 58
  53 58 0 1 2 2 1 1 0 0
  1
           1
         6.72900E-01
  1
# 4 Block-block 54 - 59
  54 59 0 1 2 2 1 1 0 0
   1
            1
   1
          5.26450E-01
# 5 Block-block 55 - 60
  55 60 0 1 2 2 1 1 0 0
   1
           1
   1
          5.43720E-01
```

```
# 6 Block-block 56 - 118
 56 118 0 1 2 2 1 1 0 0
 1 1
1 8.18380E-01
# 7 Block-block 57 - 119
  57 119 0 1 2 2 1 1 0 0
 1 1
1 4.08940E-01
# 8 Block-block 58 - 120
  58 120 0 1 2 2 1 1 0 0
  1 1
 1 6.72900E-01
# 9 Block-block 59 - 121
 59 121 0 1 2 2 1 1 0 0
 1 1
 1 5.26450E-01
# 10 Block-block 60 - 122
 60 122 0 1 2 2 1 1 0 0
  1 1
 1 5.43720E-01
# BETWEEN CONCRETE BACKFILL AND WALL BETWEEN WASTE AND GRAVEL BACKFILL
(PASSAGE)
# 11 Block-block 56 - 61
 56 61 0 1 0 1 1 1 0 0
  1 1
 1
         1.78630E-01
# BETWEEN CONCRETE BACKFILL AND CONCRETE BLOCKS ABOVE THE WASTE
# 12 Block-block 56 - 64
 56 64 0 1 0 2 1 1 0 0
 1 1
1 6.39750E-01
# 13 Block-block 57 - 67
 57 67 0 1 0 2 1 1 0 0
 1 1
1 3.18530E-01
# 14 Block-block 58 - 70
 58 70 0 1 0 2 1 1 0 0
 1 1
1 4.31200E-01
# 15 Block-block 59 - 73
  59 73 0 1 0 2 1 1 0 0
 1 1
1 2.56310E-01
# 16 Block-block 60 - 76
  60 76 0 1 0 2 1 1 0 0
  1 1
1 2.28320E-01
# BETWEEN CONCRETE BACKFILL AND CONCRETE BELOW THE WASTE
# 17 Block-block 56 - 79
 56 79 0 1 0 2 1 1 0 0
 1 1
-1 5.52740E-01
# 18 Block-block 57 - 80
 57 80 0 1 0 2 1 1 0 0
 1 1
-1 2.59470E-01
# 19 Block-block 58 - 81
  58 81 0 1 0 2 1 1 0 0
  1 1
-1 5.93020E-01
# 20 Block-block 59 - 82
  59 82 0 1 0 2 1 1 0 0
  1 1
-1 2.12730E-01
```

```
1
 1.6607E-01
# 35 Block-block 58 - 115
  58 115 0 1 2 2 0 0 0 1
  1 1
         1.60800E-02
  -1
 1
 1 6607E-01
# BETWEEN CONCRETE BACKFILL AND FICTITIOUS BLOCK
# 36 Block-block 58 - 116
  58 116 0 1 2 2 0 0 0 1
  1 1
  1
        2.41700E-01
 1
 1.6607E-01
# 37 Block-block 59 - 116
 59 116 0 1 2 2 0 0 0 1
  1 1
  -1
         2.41700E-01
 1
 1.6607E-01
# BETWEEN CONCRETE BACKFILL AND FICTITIOUS BLOCK
# 38 Block-block 59 - 117
  59 117 0 1 2 2 0 0 0 1
  1
           1
  1
        2.70140E-01
 1
 1.6607E-01
# 39 Block-block 60 - 117
  60 117 0 1 2 2 0 0 0 1
  1 1
  - 1
        2.70140E-01
 1
 1 6607E-01
# BETWEEN CONCRETE BACKFILL AND WALL BETWEEN WASTE AND GRAVEL
BACKFILL (LOADING ZONE)
# 40 Block-block 60 - 123
  60 123 0 1 2 1 1 1 0 0
  1 1
1 3.15400E-01
# WITHIN WALL BETWEEN CONCRETE BACKFILL AND GRAVEL BACKFILL (PASSAGE)
# 41 Block-block 61 - 62
  61 62 0 1 2 2 1 1 0 0
  1 1
1 1.78630E-01
# BETWEN WALL BETWEEN CONCRETE BACKFILL AND GRAVEL BACKFILL (PASSAGE)
AND FICTITIOUS BLOCK
# 42 Block-block 62 - 63
  62 63 0 1 2 2 1 0 0 0
 1 1
1 1.78630E-01
# BETWEEN FICTITIOUS BLOCK AND GRAVEL BACKFILL (PASSAGE)
# 43 Block-block 63 - 126
  63 126 0 1 2 2 0 1 0 0
  1 1
1 1.78630E-01
# WITHIN CONCRETE ABOVE THE WASTE (TRANSVERSE THE MAIN AXIS OF
TUNNEL)
# 44 Block-block 64 - 65
  64 65 0 1 1 1 1 1 0 0
 1 1
1 6.39750E-01
# 45 Block-block 65 - 66
```

```
65 66 0 1 1 1 1 1 0 0
  1 1
  1 6.39750E-01
# 46 Block-block 67 - 68
  67 68 0 1 1 1 1 1 0 0
        1
  1
  1 3.18530E-01
# 47 Block-block 68 - 69
  68 69 0 1 1 1 1 1 0 0
  1 1
  1 3.18530E-01
# 48 Block-block 70 - 71
  70 71 0 1 1 1 1 1 0 0
  1 1
  1 4.31200E-01
# 49 Block-block 71 - 72
  71 72 0 1 1 1 1 1 0 0
  1 1
  1 4.31200E-01
# 50 Block-block 73 - 74
  73 74 0 1 1 1 1 1 0 0
  1 1
       2.56310E-01
  1
# 51 Block-block 74 - 75
  74 75 0 1 1 1 1 1 0 0
  1
         1
  1
        2.56310E-01
# 52 Block-block 76 - 77
  76 77 0 1 1 1 1 1 0 0
  1
        1
  1
       2.28320E-01
# 53 Block-block 77 - 78
  77 78 0 1 1 1 1 1 0 0
        1
  1
  1
        2.28320E-01
# WITHIN CONCRETE ABOVE THE WASTE (ALONG THE MAIN AXIS OF TUNNEL)
# 54 Block-block 64 - 67
# 55 Block-block 65 - 68
# 56 Block-block 66 - 69
# 57 Block-block 67 - 70
# 58 Block-block 68 - 71
# 59 Block-block 69 - 72
# 60 Block-block 70 - 73
# 61 Block-block 71 - 74
# 62 Block-block 72 - 75
# 63 Block-block 73 - 76
# 64 Block-block 74 - 77
# 65 Block-block 75 - 78
# BETWEEN CONCRETE ABOVE THE WASTE AND GRAVEL IN TOP OF TUNNEL
# 66 Block-block 66 - 129
  66 129 0 1 1 0 1 1 0 0
  1 1
1 6.39750E-01
# 67 Block-block 69 - 132
  69 132 0 1 1 0 1 1 0 0
  1 1
         3.18530E-01
  1
# 68 Block-block 72 - 135
  72 135 0 1 1 0 1 1 0 0
  1
        1
  1
       4.31200E-01
# 69 Block-block 75 - 138
  75 138 0 1 1 0 1 1 0 0
  1
         1
```

```
- 1
        1 02480E-01
# 143 Block-block 89 - 150
  89 150 0 1 2 1 0 1 0 0
  1
         1
  -1 1.17330E-01
# 144 Block-block 92 - 151
 92 151 0 1 2 1 0 1 0 0
  1
         1
  -1 1.06000E-02
# 145 Block-block 95 - 152
  95 152 0 1 2 1 0 1 0 0
  1
         1
  -1 3.14400E-02
# 146 Block-block 98 - 153
 98 153 0 1 2 1 0 1 0 0
  1
         1
  - 1
        3.18900E-02
# WITHIN FICTITIOUS BLOCK (TRANSVERSE TO THE MAIN AXIS OF TUNNEL)
# 147 Block-block 99 - 100
 99 100 0 1 2 2 0 0 0 0
  1
         1
  -1 8.88300E-02
# 148 Block-block 100 - 101
 100 101 0 1 2 2 0 0 0 0
  1
          1
  -1
         8.88300E-02
# 149 Block-block 102 - 103
 102 103 0 1 2 2 0 0 0 0
  1
          1
  -1
          3.21400E-02
# 150 Block-block 103 - 104
 103 104 0 1 2 2 0 0 0 0
          1
  1
  - 1
         3.21400E-02
# 151 Block-block 105 - 106
 105 106 0 1 2 2 0 0 0 0
  1
          1
  - 1
         5.32000E-02
# 152 Block-block 106 - 107
  106 107 0 1 2 2 0 0 0 0
  1
         1
        5.32000E-02
  -1
# 153 Block-block 108 - 109
  108 109 0 1 2 2 0 0 0 0
  1 1
  -1
         4.05800E-02
# 154 Block-block 109 - 110
  109 110 0 1 2 2 0 0 0 0
  1 1
  - 1
         4.05800E-02
# 155 Block-block 111 - 112
 111 112 0 1 2 2 0 0 0 0
  1 1
-1 3.78100E-02
# 156 Block-block 112 - 113
 112 113 0 1 2 2 0 0 0 0
  1.
         1
         3.78100E-02
  -1
# WITHIN FICTITIOUS BLOCK (ALONG THE MAIN AXIS OF TUNNEL)
# 157 Block-block 99 - 102
# 158 Block-block 100 - 103
# 159 Block-block 101 - 104
# 160 Block-block 102 - 105
# 161 Block-block 103 - 106
```

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# 162 Block-block 104 - 107
# 163 Block-block 105 - 108
# 164 Block-block 106 - 109
# 165 Block-block 107 - 110
# 166 Block-block 108 - 111
# 167 Block-block 109 - 112
# 168 Block-block 110 - 113
# BETWEEN FICTITIOUS BLOCK AND CONCRETE BACKFILL ON RIGHT SIDE OF THE
WASTE
# 169 Block-block 101 - 154
  101 154 0 1 2 1 0 1 0 0
  1
        1
  -1 8.88300E-02
# 170 Block-block 104 - 155
  104 155 0 1 2 1 0 1 0 0
  1
          1
        3.21400E-02
  - 1
# 171 Block-block 107 - 156
  107 156 0 1 2 1 0 1 0 0
  1
          1
  -1 5.32000E-02
# 172 Block-block 110 - 157
  110 157 0 1 2 1 0 1 0 0
  1
          1
  -1
        4.05800E-02
# 173 Block-block 113 - 158
  113 158 0 1 2 1 0 1 0 0
  1
          1
          3.78100E-02
  - 1
# WITHIN WALL BETWEEN CONCRETE BACKFILL AND GRAVEL BACKFILL (LOADING
ZONE)
# 174 Block-block 123 - 124
  123 124 0 1 2 2 1 1 0 0
  1 1
  1
        3.15400E-01
# BETWEN WALL BETWEEN CONCRETE BACKFILL AND GRAVEL BACKFILL (LOADING
ZONE) AND FICTITIOUS BLOCK
# 175 Block-block 124 - 125
  124 125 0 1 2 2 1 0 0 0
  1 1
  1
        3.15400E-01
# BETWEEN FICTITIOUS BLOCK AND GRAVEL BACKFILL (LOADING ZONE)
# 176 Block-block 125 - 159
  125 159 0 1 2 2 0 1 0 0
  1
         1
  1
        3.15400E-01
# WITHIN GRAVEL BACKFILL (PASSAGE)
# 177 Block-block 126 - 127
  126 127 0 1 2 2 1 1 0 0
  1 1
        9.30500E-02
  - 1
# 178 Block-block 127 - 128
  127 128 0 1 2 2 1 1 0 0
  1 1
        6.68400E-02
  - 1
# BETWEEN GRAVEL BACKFILL (PASSAGE) AND GRAVEL IN TOP OF TUNNEL
# 179 Block-block 126 - 129
  126 129 0 1 0 2 1 1 0 0
  1
          1
  1
         1.15893E-01
# 180 Block-block 126 - 130
  126 130 0 1 0 2 1 1 0 0
  1
         1
   1
         1.15893E-01
```

```
# 181 Block-block 126 - 131
 126 131 0 1 0 2 1 1 0 0
  1 1
 1 1.15893E-01
# BETWEEN GRAVEL BACKFILL (PASSAGE) AND GRAVEL IN BOTTOM OF TUNNEL
# 182 Block-block 126 - 144
 126 144 0 1 1 2 1 1 0 0
 1
         1
 -1 4.30820E-02
# BETWEEN GRAVEL BACKFILL (PASSAGE) AND CONCRETE BACKFILL ON LEFT
SIDE OF STRUCTURE
# 183 Block-block 126 - 149
 126 149 0 1 0 2 1 1 0 0
  1 1
  -1 1.10400E-03
# 184 Block-block 126 - 162
 126 162 0 1 0 2 1 1 0 0
  1 1
  -1 1.10400E-03
# 185 Block-block 126 - 163
 126 163 0 1 0 2 1 1 0 0
  1
         1
  -1 1.10400E-03
# BETWEEN GRAVEL BACKFILL (PASSAGE) AND CONCRETE BACKFILL ON RIGHT
SIDE OF STRUCTURE
# 186 Block-block 126 - 154
 126 154 0 1 0 2 1 1 0 0
  1 1
  -1 1.13067E-03
# 187 Block-block 126 - 172
 126 172 0 1 0 2 1 1 0 0
  1 1
  0
       1.13067E-03
# 188 Block-block 126 - 173
 126 173 0 1 0 2 1 1 0 0
  1 1
  -1 1.13067E-03
# WITHIN GRAVEL IN TOP OF THE TUNNEL (TRANSVERSE TO THE MAIN AXIS OF
TUNNEL)
# 189 Block-block 129 - 130
 129 130 0 1 0 0 1 1 0 0
  1 1
       5.89643E-01
 1
# 190 Block-block 130 - 131
 130 131 0 1 0 0 1 1 0 0
 1 1
1 4.46787E-01
# 191 Block-block 132 - 133
 132 133 0 1 0 0 1 1 0 0
 1 1
1 2.82580E-01
# 192 Block-block 133 - 134
 133 134 0 1 0 0 1 1 0 0
 1 1
1 1.52660E-01
# 193 Block-block 135 - 136
 135 136 0 1 0 0 1 1 0 0
  1 1
1 2.07099E+00
# 194 Block-block 136 - 137
 136 137 0 1 0 0 1 1 0 0
 1 1
1 3.50146E+00
# 195 Block-block 138 - 139
```

```
138 139 0 1 0 0 1 1 0 0
 1 1
  1 2.17500E-01
# 196 Block-block 139 - 140
 139 140 0 1 0 0 1 1 0 0
  1
        1
  1 1.42759E-01
# 197 Block-block 141 - 142
 141 142 0 1 0 0 1 1 0 0
  1 1
  1 2.22601E-01
# 198 Block-block 142 - 143
 142 143 0 1 0 0 1 1 0 0
  1 1
       1.83233E-01
 1
# WITHIN GRAVEL IN TOP OF THE TUNNEL (ALONG THE MAIN AXIS OF TUNNEL)
# 199 Block-block 129 - 132
 129 132 0 1 2 2 1 1 0 0
 1 1
 1
       3.35100E-01
# 200 Block-block 130 - 133
 130 133 0 1 2 2 1 1 0 0
  1
         1
  1
        3.35100E-01
# 201 Block-block 131 - 134
 131 134 0 1 2 2 1 1 0 0
  1
         1
  1
        3.35100E-01
# 202 Block-block 132 - 135
 132 135 0 1 2 2 1 1 0 0
  1
         1
  1
         5.84867E-01
# 203 Block-block 133 - 136
 133 136 0 1 2 2 1 1 0 0
  1
         1
  1
        5.84867E-01
# 204 Block-block 134 - 137
 134 137 0 1 2 2 1 1 0 0
         1
  1
  1
       5.84867E-01
# 205 Block-block 135 - 138
 135 138 0 1 2 2 1 1 0 0
  1 1
  -1 4.08700E-01
# 206 Block-block 136 - 139
 136 139 0 1 2 2 1 1 0 0
  1 1
  - 1
       4.08700E-01
# 207 Block-block 137 - 140
 137 140 0 1 2 2 1 1 0 0
  1 1
  -1
       4.08700E-01
# 208 Block-block 138 - 141
 138 141 0 1 2 2 1 1 0 0
  1 1
-1 2.99663E-01
# 209 Block-block 139 - 142
 139 142 0 1 2 2 1 1 0 0
  1 1
1 2.99663E-01
# 210 Block-block 140 - 143
 140 143 0 1 2 2 1 1 0 0
  1 1
  -1
         2.99663E-01
```

```
# BETWEEN GRAVEL IN TOP OF THE TUNNEL AND CONCRETE ON LEFT SIDE OF
THE TUNNEL
# 211 Block-block 129 - 149
 129 149 0 1 1 0 1 1 0 0
 1 1
-1 1.59743E-02
# 212 Block-block 129 - 162
  129 162 0 1 1 0 1 1 0 0
         1
  1
  -1 1.59743E-02
# 213 Block-block 129 - 163
  129 163 0 1 1 0 1 1 0 0
  1
         1
  -1
       1.59743E-02
# 214 Block-block 132 - 150
 132 150 0 1 1 0 1 1 0 0
 1
         1
  -1 2.47607E-02
# 215 Block-block 132 - 164
 132 164 0 1 1 0 1 1 0 0
  1
         1
  -1 2.47607E-02
# 216 Block-block 132 - 165
 132 165 0 1 1 0 1 1 0 0
  1
          1
  -1
         2.47607E-02
# 217 Block-block 135 - 151
 135 151 0 1 1 0 1 1 0 0
  1
          1
  -1
         3.21630E-02
# 218 Block-block 135 - 166
 135 166 0 1 1 0 1 1 0 0
         1
  1
  - 1
         3.21630E-02
# 219 Block-block 135 - 167
 135 167 0 1 1 0 1 1 0 0
  1
         1
  -1
        3.21630E-02
# 220 Block-block 138 - 152
 138 152 0 1 1 0 1 1 0 0
  1
         1
        5.77700E-03
  -1
# 221 Block-block 138 - 168
  138 168 0 1 1 0 1 1 0 0
  1 1
         5.77700E-03
  -1
# 222 Block-block 138 - 169
  138 169 0 1 1 0 1 1 0 0
  1 1
  - 1
         5.77700E-03
# 223 Block-block 141 - 153
 141 153 0 1 1 0 1 1 0 0
 1
         1
  -1
        5.45700E-03
# 224 Block-block 141 - 170
 141 170 0 1 1 0 1 1 0 0
  1.
         1
         5.45700E-03
  - 1
# 225 Block-block 141 - 171
 141 171 0 1 1 0 1 1 0 0
  1
         1
  - 1
         5.45700E-03
# BETWEEN GRAVEL IN TOP OF THE TUNNEL AND CONCRETE ON RIGHT SIDE OF
THE TUNNEL
```

```
1
         1
 -1 8.50300E-03
# 242 Block-block 145 - 146
 145 146 0 1 2 2 1 1 0 0
 1
         1
 -1 2.44320E-02
# 243 Block-block 146 - 147
 146 147 0 1 2 2 1 1 0 0
 1
        1
 1 3.13910E-02
# 244 Block-block 147 - 148
 147 148 0 1 2 2 1 1 0 0
 1 1
 1
         6.61900E-03
# WITHIN CONCRETE BACKFILL ON LEFT SIDE OF THE TUNNEL (ALONG THE MAIN
AXIS OF TUNNEL)
# 245 Block-block 149 - 150
 149 150 0 1 2 2 1 1 0 0
 1
         1
 -1 4.99333E-04
# 246 Block-block 150 - 151
 150 151 0 1 2 2 1 1 0 0
 1
         1
  1
        4.20000E-05
# 247 Block-block 151 - 152
 151 152 0 1 2 2 1 1 0 0
  1
          1
  1
        4.98667E-04
# 248 Block-block 152 - 153
 152 153 0 1 2 2 1 1 0 0
  1
         1
  1
        1.43333E-04
# 249 Block-block 162 - 164
 162 164 0 1 2 2 1 1 0 0
  1
         1
  - 1
        4.99333E-04
# 250 Block-block 163 - 165
 163 165 0 1 2 2 1 1 0 0
  1
         1
  -1
        4.99333E-04
# 251 Block-block 164 - 166
 164 166 0 1 2 2 1 1 0 0
  1 1
 1
       4.20000E-05
# 252 Block-block 165 - 167
 165 167 0 1 2 2 1 1 0 0
  1 1
 1
       4.20000E-05
# 253 Block-block 166 - 168
 166 168 0 1 2 2 1 1 0 0
  1 1
       4.98667E-04
 1
# 254 Block-block 167 - 169
 167 169 0 1 2 2 1 1 0 0
 1 1
1 4.98667E-04
# 255 Block-block 168 - 170
 168 170 0 1 2 2 1 1 0 0
 1 L
1.43333E-04
# 256 Block-block 169 - 171
 169 171 0 1 2 2 1 1 0 0
 1 1
1 1.43333E-04
```
```
# 272 Block-block 173 - 175
 173 175 0 1 2 2 1 1 0 0
 1
         1
 -1 1.07667E-04
# 273 Block-block 174 - 176
 174 176 0 1 2 2 1 1 0 0
 1
         1
 -1 4.66667E-04
# 274 Block-block 175 - 177
 175 177 0 1 2 2 1 1 0 0
  1 1
 -1 4.66667E-04
# 275 Block-block 176 - 178
 176 178 0 1 2 2 1 1 0 0
 1
        1
 1
       5.00333E-04
# 276 Block-block 177 - 179
 177 179 0 1 2 2 1 1 0 0
 1
         1
       5.00333E-04
 1
# 277 Block-block 178 - 180
 178 180 0 1 2 2 1 1 0 0
 1
         1
  1
       1.66667E-04
# 278 Block-block 179 - 181
 179 181 0 1 2 2 1 1 0 0
  1
         1
        1.66667E-04
  1
# WITHIN CONCRETE BACKFILL ON RIGHT SIDE OF THE TUNNEL (TRANSVERSE TO
THE MAIN AXIS OF TUNNEL)
# 279 Block-block 154 - 172
 154 172 0 1 1 1 1 1 0 0
         1
 1
 -1 9.18317E-02
# 280 Block-block 172 - 173
 172 173 0 1 1 1 1 1 0 0
  1
         1
 -1
        9.48353E-02
# 281 Block-block 155 - 174
 155 174 0 1 1 1 1 1 0 0
  1
         1
       3.24783E-02
  -1
# 282 Block-block 174 - 175
 174 175 0 1 1 1 1 1 0 0
  1 1
  -1
        3.28127E-02
# 283 Block-block 156 - 176
 156 176 0 1 1 1 1 1 0 0
  1 1
  - 1
        8.28490E-02
# 284 Block-block 176 - 177
 176 177 0 1 1 1 1 1 0 0
 1 1
-1 1.12499E-01
# 285 Block-block 157 - 178
 157 178 0 1 1 1 1 1 0 0
 1 1
-1 4.16230E-02
# 286 Block-block 178 - 179
 178 179 0 1 1 1 1 1 0 0
  1 1
-1 4.26680E-02
# 287 Block-block 158 - 180
 158 180 0 1 1 1 1 1 0 0
```

```
1
          1
  -1 3.96943E-02
# 288 Block-block 180 - 181
  180 181 0 1 1 1 1 1 0 0
  1 1
  -1 4.15787E-02
# WITHIN GRAVEL IN LOADING ZONE
# 289 Block-block 159 - 160
  159 160 0 1 2 2 1 1 0 0
  1 1
  -1 2.37257E-01
# 290 Block-block 160 - 161
  160 161 0 1 2 2 1 1 0 0
  1 1
  -1 1.31023E-01
# BETWEEN GRAVEL IN LOADING ZONE AND GRAVEL IN TOP OF TUNNEL
# 291 Block-block 159 - 141
  159 141 0 1 0 2 1 1 0 0
  1 1
        2.35607E-01
  1
# 292 Block-block 159 - 142
  159 142 0 1 0 2 1 1 0 0
  1
          1
  1
        2.35607E-01
# 293 Block-block 159 - 143
  159 143 0 1 0 2 1 1 0 0
  1
          1
         2.35607E-01
  1
# BETWEEN GRAVEL IN LOADING ZONE AND GRAVEL IN BOTTOM OF TUNNEL
# 294 Block-block 159 - 148
  159 148 0 1 0 2 1 1 0 0
  1 1
  -1
         4.17120E-02
# BETWEEN GRAVEL IN LOADING ZONE AND CONCRETE BACKFILL ON LEFT SIDE
OF TUNNEL
# 295 Block-block 159 - 153
  159 153 0 1 0 2 1 1 0 0
  1 1
  -1 9.91667E-04
# 296 Block-block 159 - 170
  159 170 0 1 0 2 1 1 0 0
  1 1
       9.91667E-04
  - 1
# 297 Block-block 159 - 171
  159 171 0 1 0 2 1 1 0 0
  1
         1
  - 1
      9.91667E-04
# BETWEEN GRAVEL IN LOADING ZONE AND CONCRETE BACKFILL ON RIGHT SIDE
OF TUNNEL
# 298 Block-block 159 - 158
  159 158 0 1 0 2 1 1 0 0
  1 1
      1.07800E-03
  - 1
# 299 Block-block 159 - 180
  159 180 0 1 0 2 1 1 0 0
  1 1
  -1
         1.07800E-03
# 300 Block-block 159 - 181
  159 181 0 1 0 2 1 1 0 0
  1 1
-1 1.07800E-03
```

```
#
  BLA CALCULATIONS
#
  Problem Codification: BS1
# Nuclide(s): C-14 org
# Problem Description:
# PROPER VERSION INPUT DATA FORMAT
  ITYPE NGROUP NSINKS
                           IWFLOW
                                   NSOURC
                                            NBSIZE
                                                    ISPSOL
#
         1
                  0
                           1
                                    5
                                            0
                                                     0
  1
# Flow stream conditions
# NINFLW NUTFLW
                            % if TWFLOW = 1
  7
          7
# Inflow conditions (one line per inflow
# QINFW NBWIN NBNWIN
                         % if IWFLOW = 1
  1.34563 1
                 1
  1.01519 2
                 1
  9.4796 3
                1
  0.66294 4
                 1
  0.15947 5
                 1
  0.1139 6
                 1
  0.60853 7
                 1
  Outflow conditions (one line per inflow)
#
  QUTFW NBWOUT NBNWOUT % if IWFLOW = 1
#
  0.7824 1
                  1
  0.4314 2
                  1
  11.33948 3
                  1
  0.2742 4
                  1
  0.1351 5
                  1
  0.05295 6
                  1
  0.36973 7
                  1
# Number of blocks connections and materials
# NBLOCK NBCOUP NMAT
  7
          6
                   2
# Data on materials (one line per material)
# Material 1 = Waste
# Material 2 = Water
# MAT
           DENSM
                      PORM
           1000.0
                      0.632
  1
           1000.0
                      1.000
  2
# Data on time series
# TINIT AGEO RAT
                           TEND
                                    NTERM
                                             LREAD
  0.00
          0.50
                1.23
                         1000.00 90
                                             0
#
# Data on numerical parameters
# EPS
       EWT
                  NLOOP
                           MSPAR
  1.00E-06 1.00E-18 3
                           1
# Data on group (one set of data per group common for all sources)
# Group No: 1
# NNCH ISPEC ISPSOL
  1 1
                0
#
# Nuclide: C-14 org
# TAU
           CSOLUB
                      AINV
  5.7300E+03 1.00E+10
                    1.4159E-05
#
```

# CIRF % if TSPEC = 1 # # MAT DEFM SKDM % one line per material 0 06307 0 000 1 0.06307 2 0.000 Data on sources NSBCHA ISEO MAT IFWATB VOLSOU FINVEN JINVEN TBLOCK ISSEO % if ISEQ = 1 # # XXX % if JINVEN = 1. One line for each group # IBSOUR % if ISEQ = 0 TWINT % if IFWATB = 1 # WBFLOW % if IFWATB = 1 and IWINT = 0 # # Waste Source 1 1 0 1 1 10069.5150 1.0000000E+00 1 4.0707965E-01 1 1 # Source 2 # 1 0 1 1 6129.2700 1.000000E+00 1 # 2.4778761E-01 2 1 # Source 3 # 1 0 1 1 3064.6350 1.000000E+00 1 # 1.2389381E-01 3 1 Source 4 # 1 0 1 1 3502.4400 1.000000E+00 1 1.4159292E-01 4 1 # # Source 5 1 0 1 1 1970.1225 1.0000000E+00 1 7.9646018E-02 5 1 Description of blocks not considered before # # Block i # NZ NY NX NBLSP MAT IQFLOW ZADIM YADIM XADIM VOLBLK % one line per compartment if NBLSP = 1 # IWINT % if IOFLOW = 1 #

```
# NODWIN NODWUT BFLOW % if IQFLOW = 1 and IWINT = 0
#
# Loading zone at S.1.
# Block 6
 1 1 1 0 2 1
 1.0000E+00 1.0000E+00
                         1.0000E+00
                                       6.9000E+02
 1
#
# Loading zone at S.9.
# Block 7
 1 1 1 0 2 1
 1.0000E+00 1.0000E+00
                         1.0000E+00
                                       2.1740E+03
 1
#
# Codifying the connections
#
# NSSCO NSBCO NBBCO
 4 2 0
#
# Connection between sources
# NABCO ICSEO LSCFW IRADD
#
                 % if LSCFW is not equal to 0
# QWATIN
#
# ISERNU
                 % if IRADD = 1
# XRADD
                 % if IRADD = 1 and ISERNU = 1 one line per group
# XRADD
                 % if IRADD = 1 and ISERNU = 0 one line per group
#
# IBSA IBSB ISEQA ISEQB % if NABCO > 1 and ICSEQ = 1
# IBSA IBSB
                         % if ICSEQ = 0 NABCO*lines
# WITHIN WASTE
#
# 1 Source-source 1 - 2
   1
                 0
                              1
                                            0
   6.242200E-01
                 2
   1
 2 Source-source 2 - 3
#
   1
                 0
                               1
                                             0
   1.207930E+00
    2
                 3
 3 Source-source 3 - 4
#
   1
                 0
                               -1
                                             0
    6.519100E-01
                 4
   3
#
  4 Source-source 4 - 5
#
   1
                 0
                               -1
                                             0
    2.631600E-01
    4
                 5
  Connection source - block
#
  IBSOU IBSB ISEQC NABCCO IRZB ICRB IPLUG
                                                     IRADD
#
```

```
# ICOMPB ICSEQB % if NABCCO > 1 and ISEQC =1
#
 TDCB
                   % if ISEOC = 0. One value for each connection.
 LSCFW OWATIN % if IFWATB(source term) = 1 and IOFLOW(block) = 1
#
  ISERNU
               % if IRADD =1
#
  XRADD
               % if IRADD = 1 and ISERNU = 1
#
#
  XRADD
               % if IRADD = 1 and ISERNU = 0. One line per group.
# BETWEEN WASTE - LOADING ZONES
# 1 Source-block 1 - 6
    1 6 0 1 2 0 0 0
#
 1
 -1 0.06094
# 2 Source-block 5 - 7
    5 7 0 1 2 0 0 0
#
 1
 -1 0.23879
```

## Appendix D: Biosphere models

Schematic figures of the biosphere models.



Figure D-1 Structure of the coastal model. The cross marks the source of radionuclides.



Figure D-2 Structure of the lake model. The cross marks the source of radionuclides.



Figure D-3 Structure of the agricultural land model. The crosses mark the source of radionuclides.



Figure D-4 Structure of the mire model. The cross marks the source of radionuclides.



Figure D-5 Structure of the well model. The cross marks the source of radionuclides.